



Current status of the liquid lithium target development

LiLiT Team
presented by S. Halfon

4th High-Power Targetry Workshop

May 3, 2011

	Mg									Al	S	
Ca	Sc	Ti	V	Cr	Mn	Fe	Co	Ni	Cu	Zn	Ga	Ge
Sr	Y	Zr	Nb	Mo	Tc	Ru	Rh	Pd	Ag	Cd	In	Sn
Ba		Hf	Ta	W	Re	Os	Ir	Pt	Au	Hg	Tl	Pb
Ra												
	La	Ce	Pr	Nd	Pm	Sm	Eu	Gd	Tb	Dy	Ho	Er

Outline

- ❖ Soreq Applied Research Accelerator Facility (SARAF) overview
- ❖ Liquid Lithium Target
 - research application and requirements (BNCT, astrophysics)
 - design features
 - lithium circulation and e-gun experiments

SARAF Accelerator

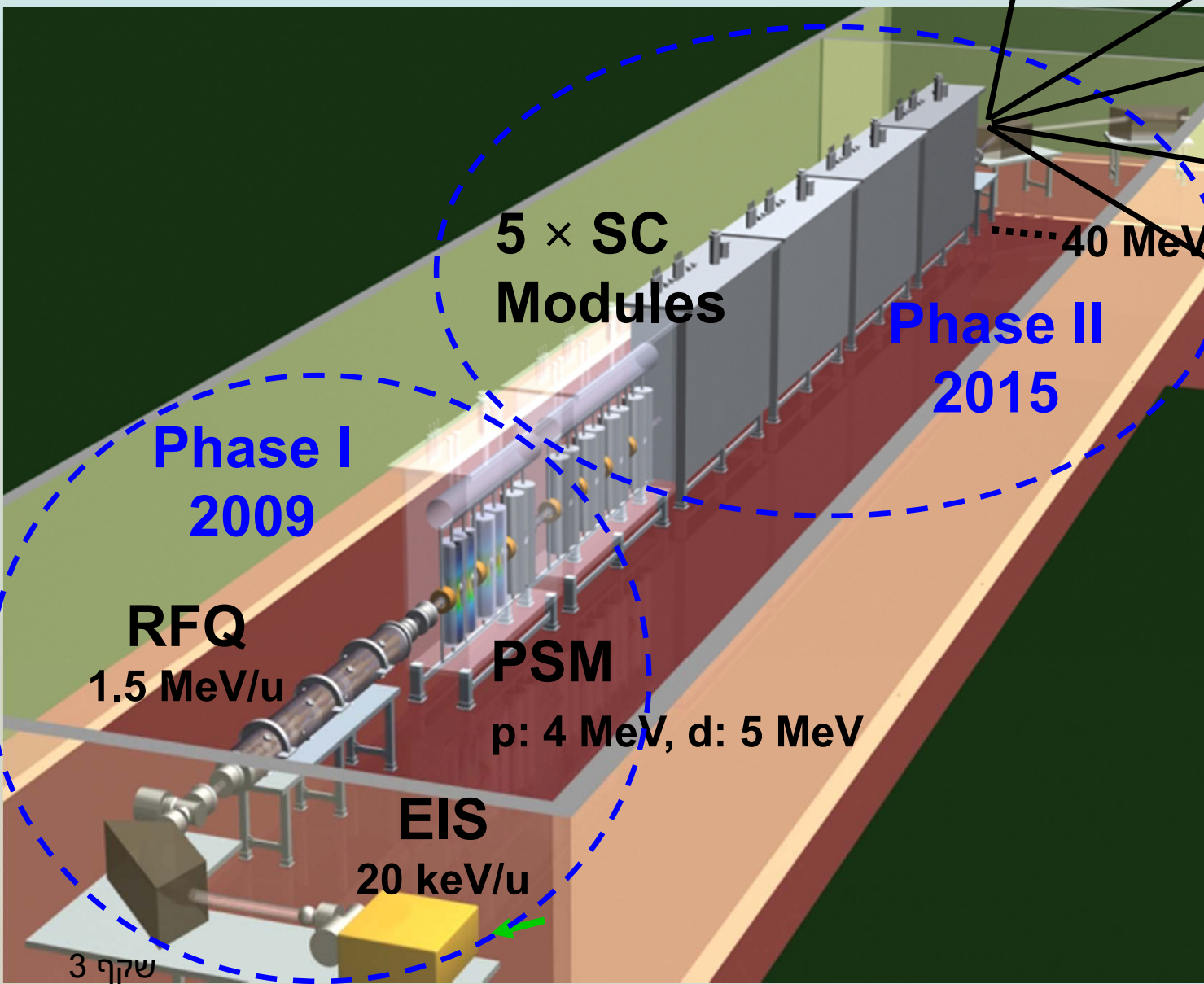
Thermal neutron radiography

Thermal neutron diffraction

Nuclear Astrophysics

Radioactive beams

Radio Pharmaceuticals



Accelerator Parameters

Parameter	Value
Ions	p / d
Energy	5 – 40 MeV
Current	0.04 – 2 mA
Maintenance	Hands-On

•Current upgradeable to 4 mA

SARAF Phase I – Upstream View

PSM

MEBT

RFQ

LEBT

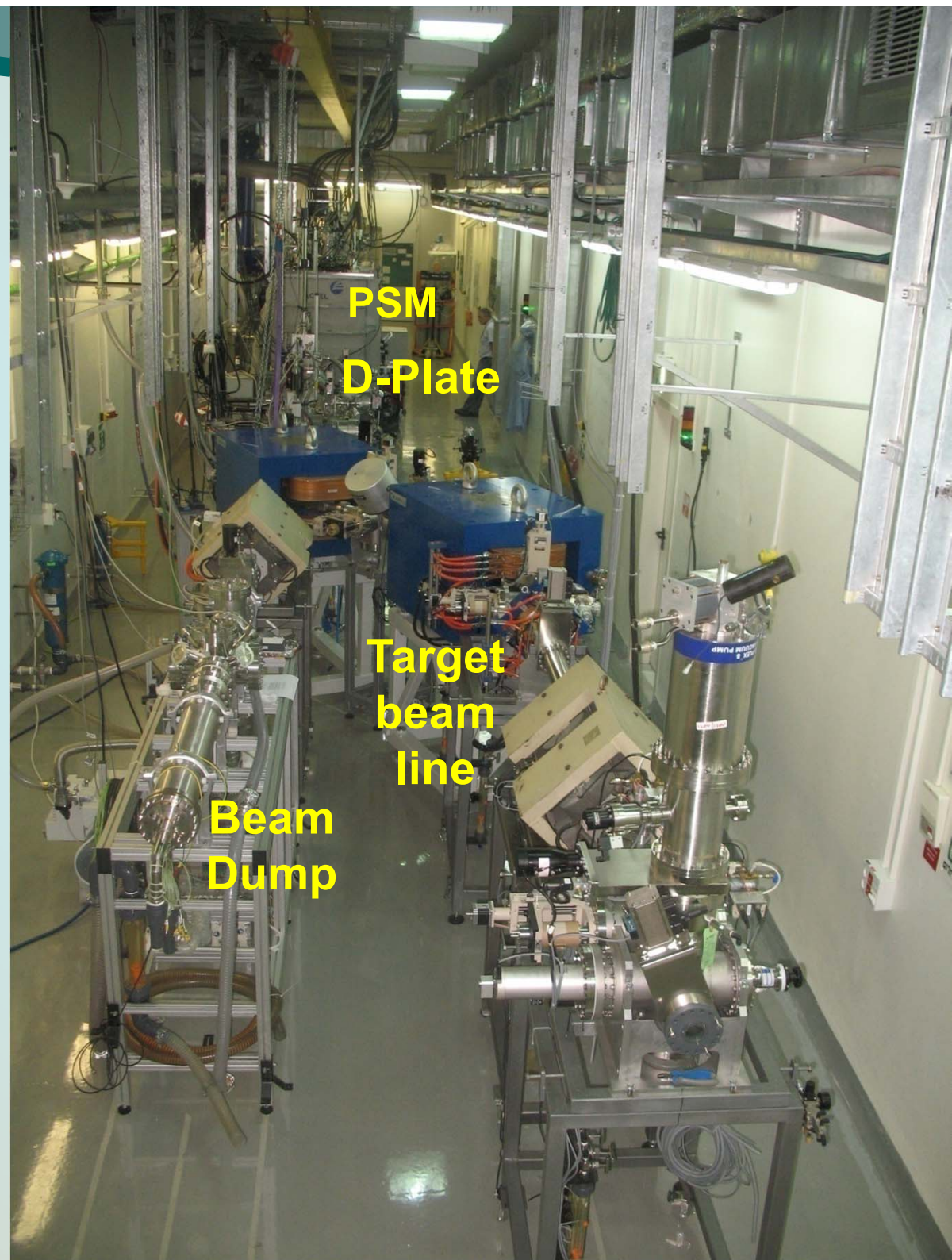
EIS

ACCEL

A. Nagler, Linac-2006
C. Piel, EPAC-2008
A. Nagler, Linac-2008
I. Mardor, PAC-2009

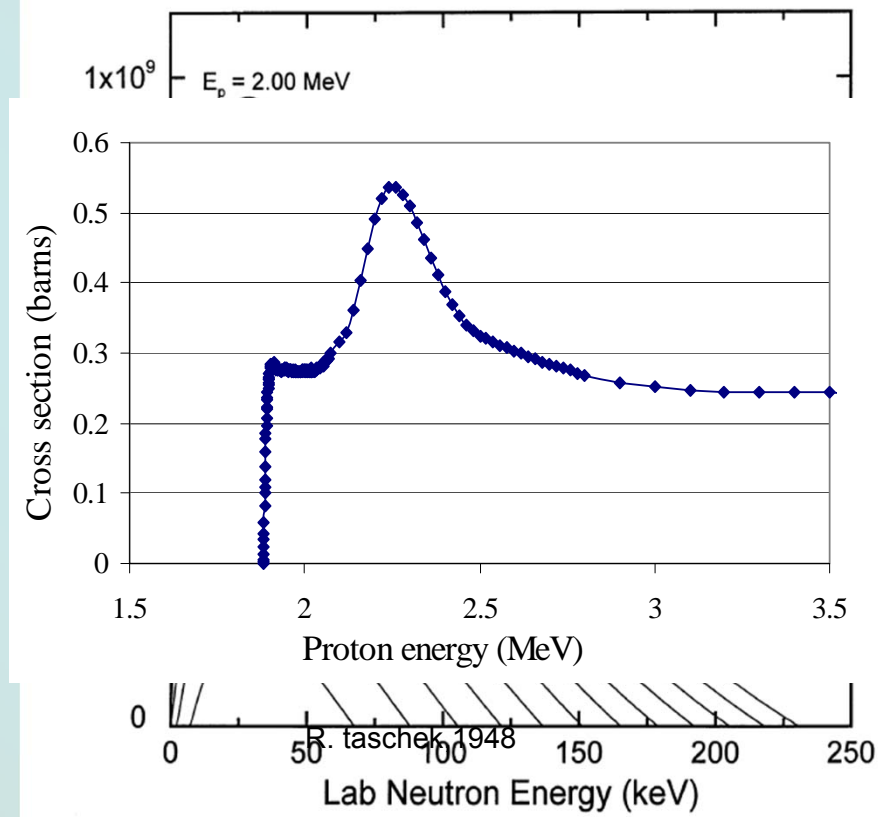
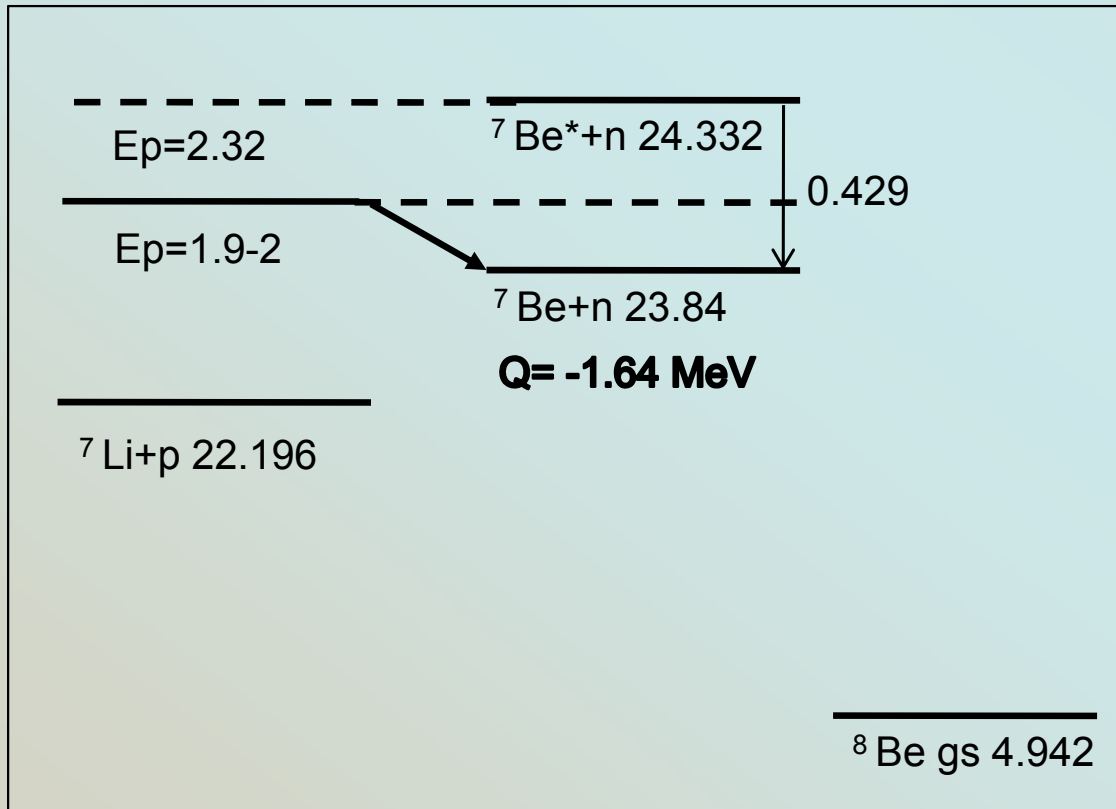
SARAF Phase I – downstream

- ❖ Commissioning of Phase-I is approaching finalization
- ❖ The current challenges include conditioning the RFQ to enable acceleration of CW deuterons
- ❖ 1 mA CW proton beam have been accelerated through the entire Phase-I up energy of 3.7 MeV
- ❖ Low duty cycle 2.5 mA deuteron beam have been accelerated to energy of 4.3 MeV



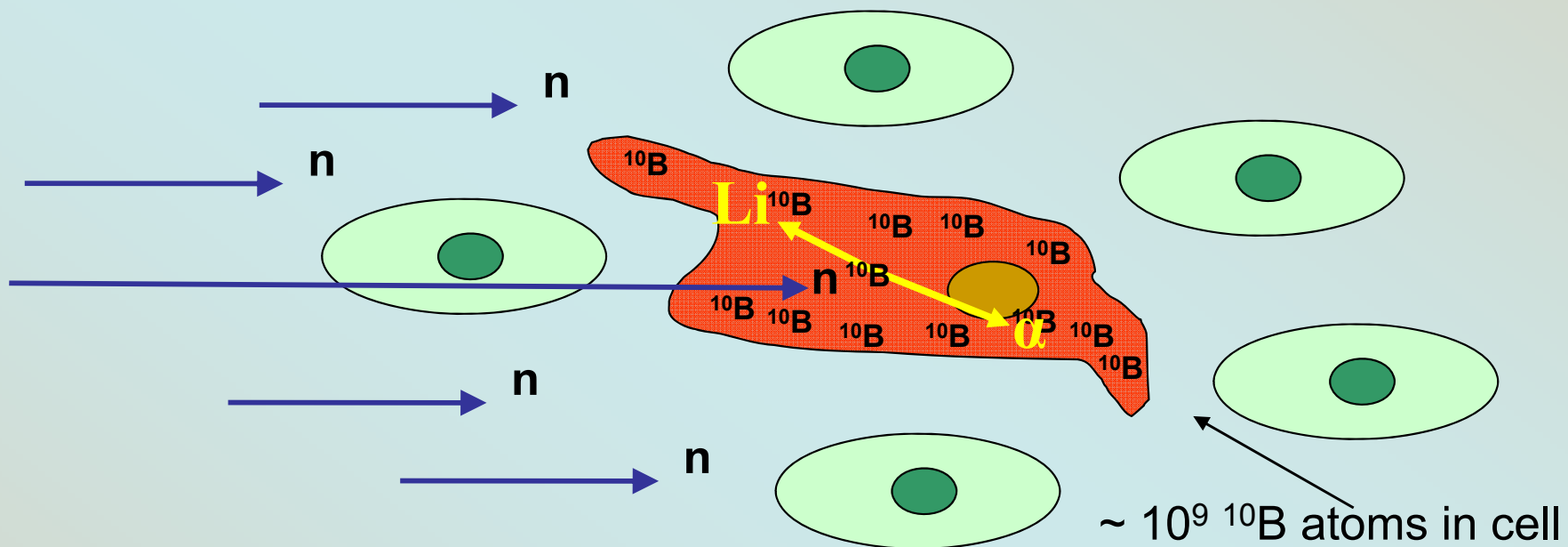
Neutron producing lithium target - ${}^7\text{Li}(p,n){}^7\text{Be}$

- $E_{thr}(p) = 1.881 \text{ MeV}$, $Q = -1.644 \text{ MeV}$.
- Produces keV-energy forward-collimated neutrons near threshold.



C.L. Lee, X.-L. Zhou, Nucl. Instr. and Meth. in Phys. Res. B 152 (1999) 1-11

Boron Neutron Capture Therapy



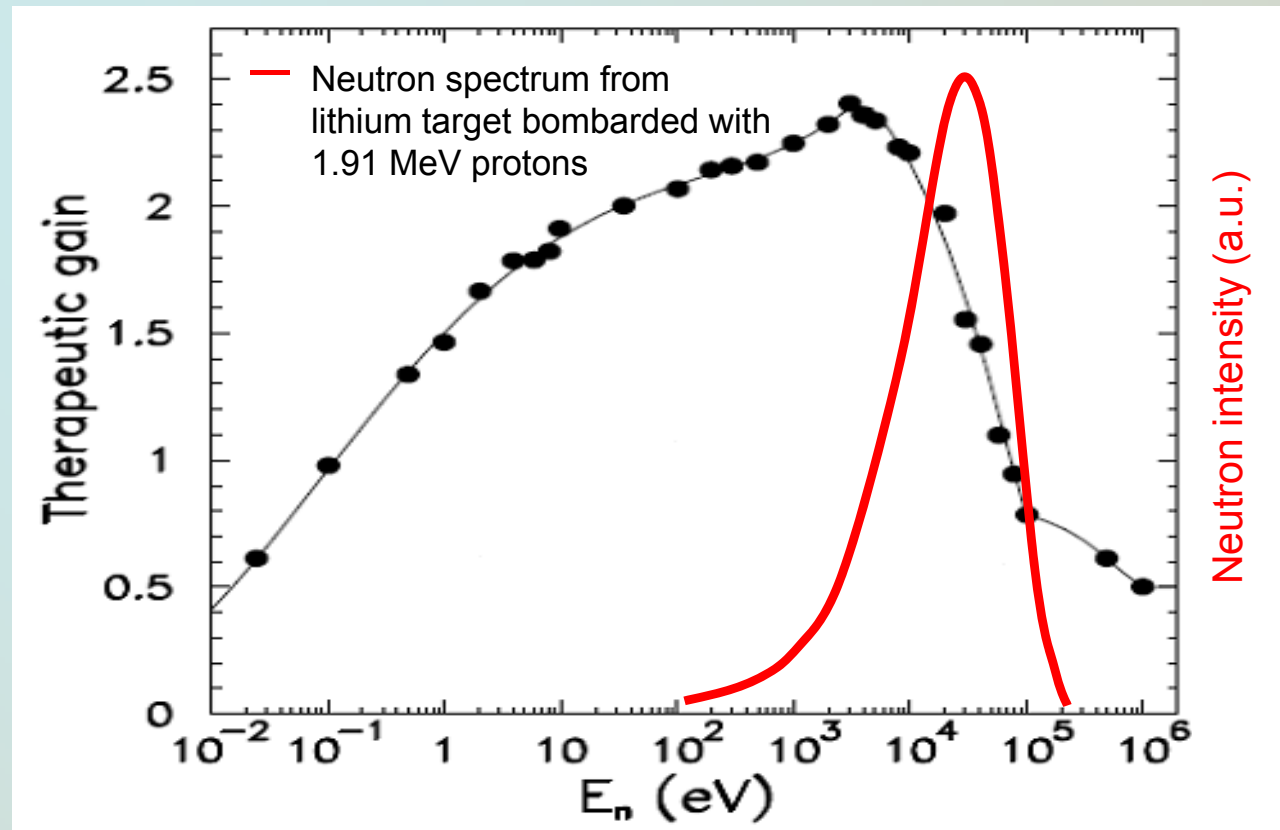
1. Selectively deliver ^{10}B to the tumor cells
2. Irradiate the target region with neutrons
3. The short range of the $^{10}\text{B}(n,\alpha)^7\text{Li}$ reaction product, 5-8 mm in tissue, restrict the dose to the boron loaded area

The neutron energy effect on therapy

Optimal Energy for deep-seated tumor: **0.5 eV – 10 keV**

Accelerator based BNCT with lithium target:

1. Produce most suitable neutrons for therapy
2. Small- in hospital
3. Good public acceptability
4. Relatively cheap

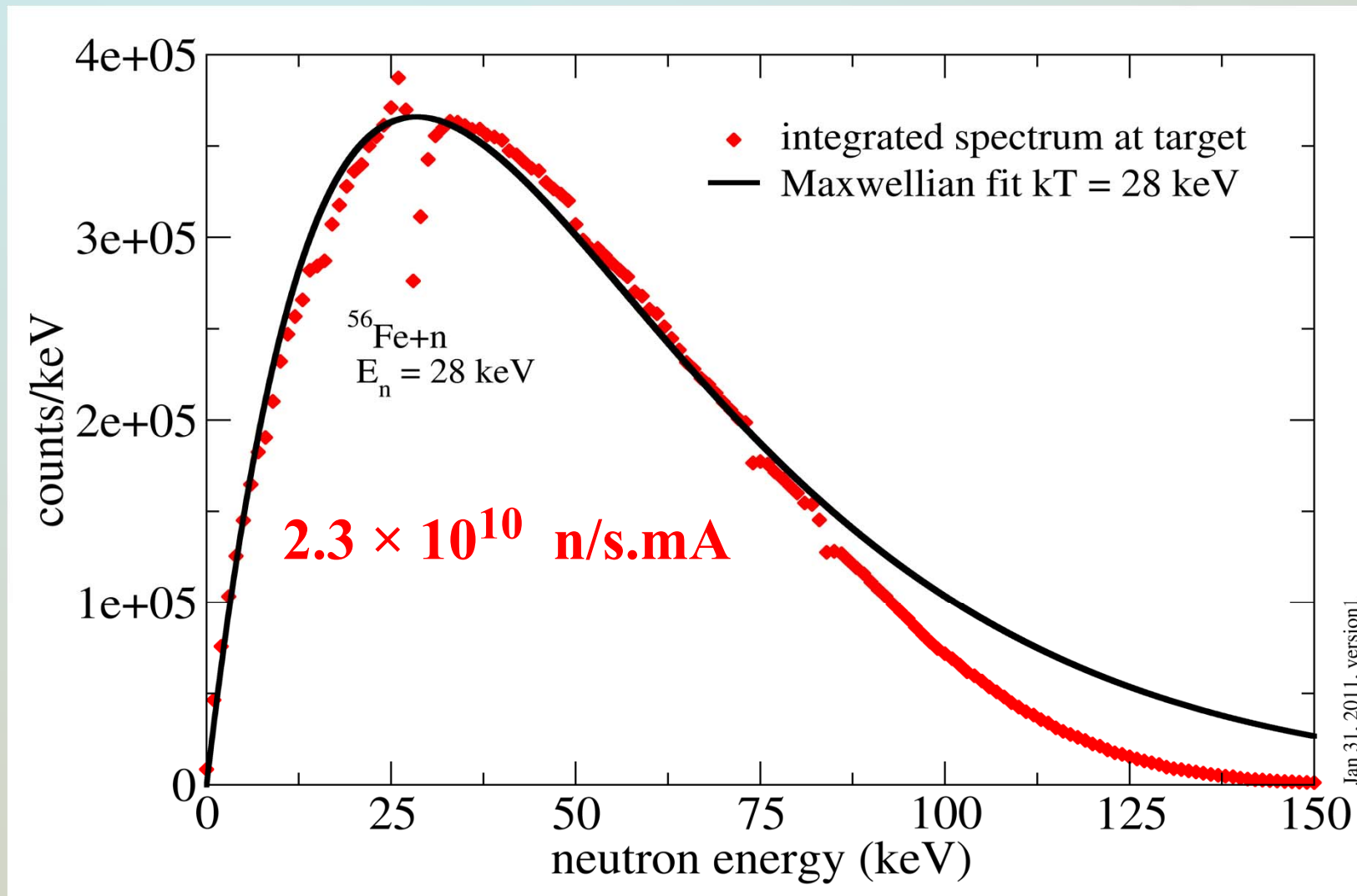


Bisceglie et. al. Phys. Med. Biol. **45** (2000) 49–58.

Neutron flux: Optimal $\approx 10^9 \text{ s}^{-1} \text{ cm}^{-2}$ on beam port ** (for ~1 hour therapy)

SARAF lithium target $> 10^{10} \text{ s}^{-1} \text{ mA}^{-1}$

Astrophysical research: at $E_p=1.91$ MeV a neutron spectrum of maxwellian with $kT= \sim 28$ keV is produced-
typical stellar neutron energy in s-proces



LiLiT full-geometry simulation (GEANT4)

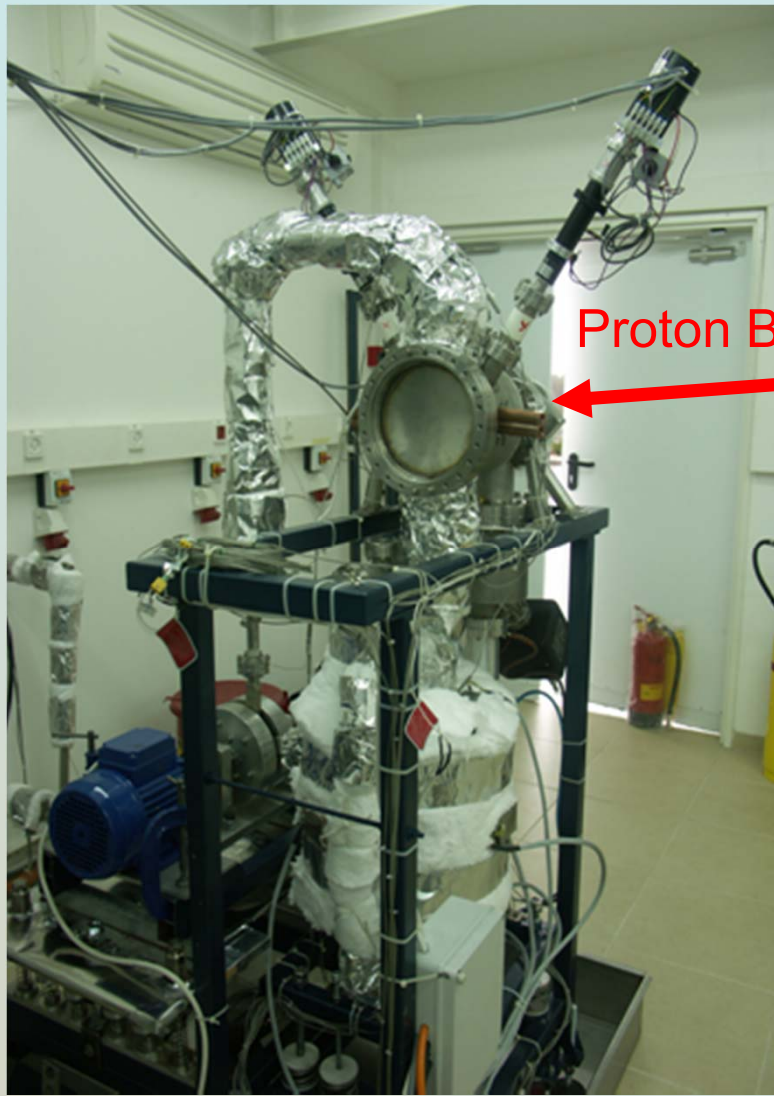
LiLiT – High flux keV neutron source

- ❖ Both researches require high neutron flux ($\sim 10^9$ n/cm²/s) hence high power Lithium Target
- ❖ 4 – 10 kW beam power (p, 2-4 mA, 1.9-2.5 MeV)
- ❖ Gaussian beam ($\sigma=2$ mm, D=12 mm)

Project	IFMIF *	SPIRAL II *	LiLiT
Reaction specification	d(40 MeV) +Li	d(40 MeV) + C	p(2 MeV) +Li
Projectile range in target (mm)	19.1	4.3	0.2
Maximum beam current (mA)	2 x 125	5	2
Beam spot on the target (cm ²)	~100	~10	~1
Beam density on the target (mA/cm ²)	2.5	0.5	>2 (peak)

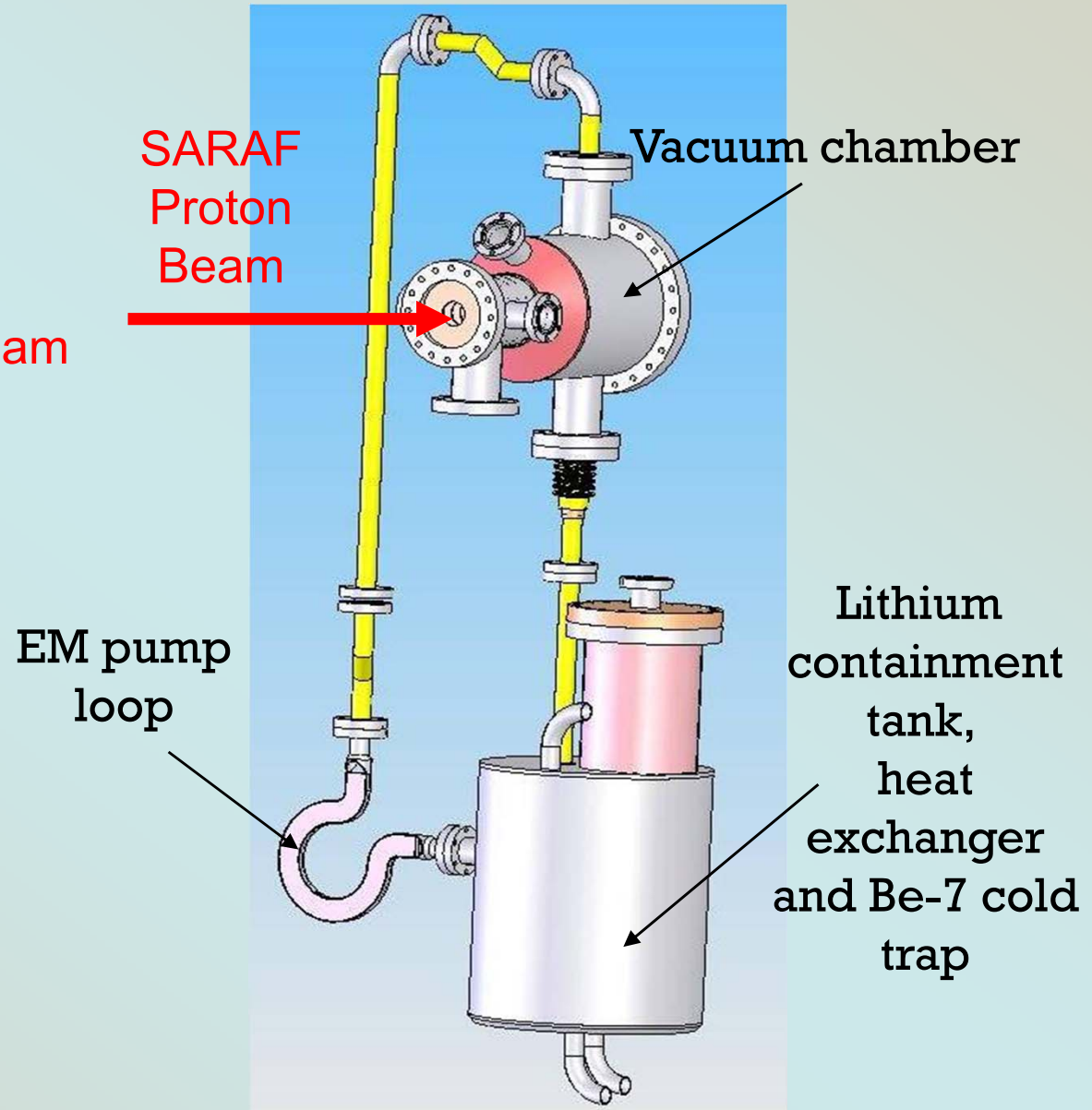
- ❖ The target should dissipate power densities of more then ~ 1 MW/cm³

Liquid lithium loop



Proton Beam

Neutron port



SARAF
Proton
Beam

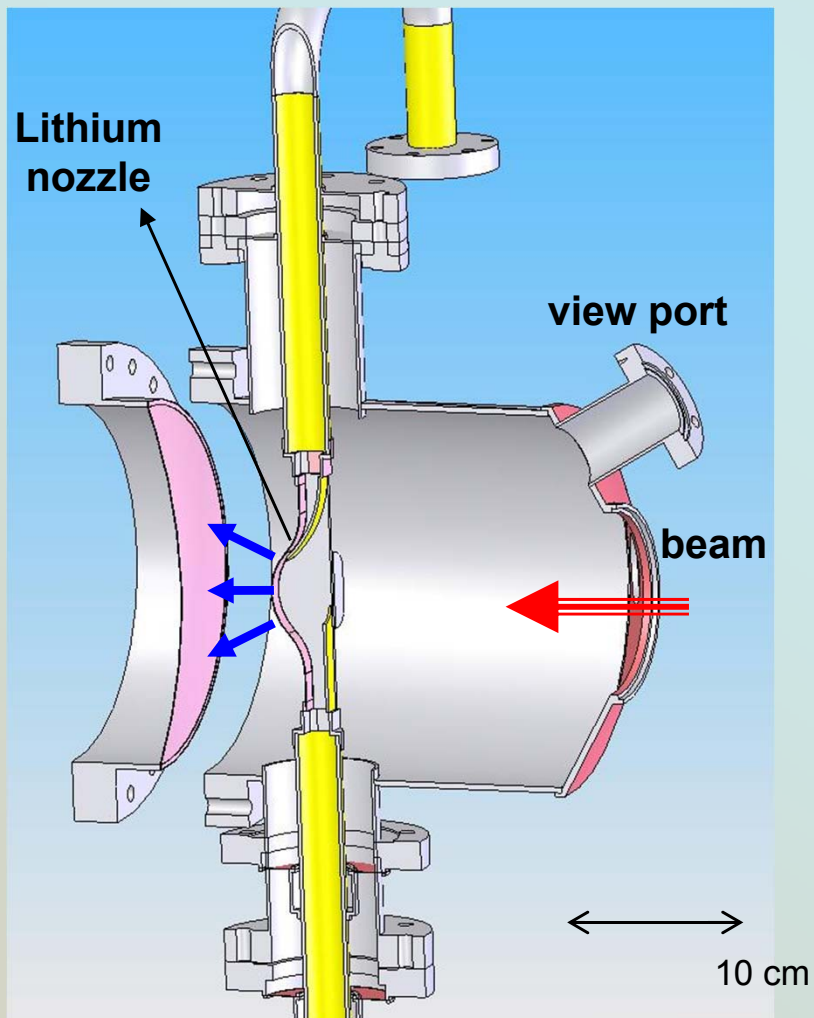
Vacuum chamber

EM pump
loop

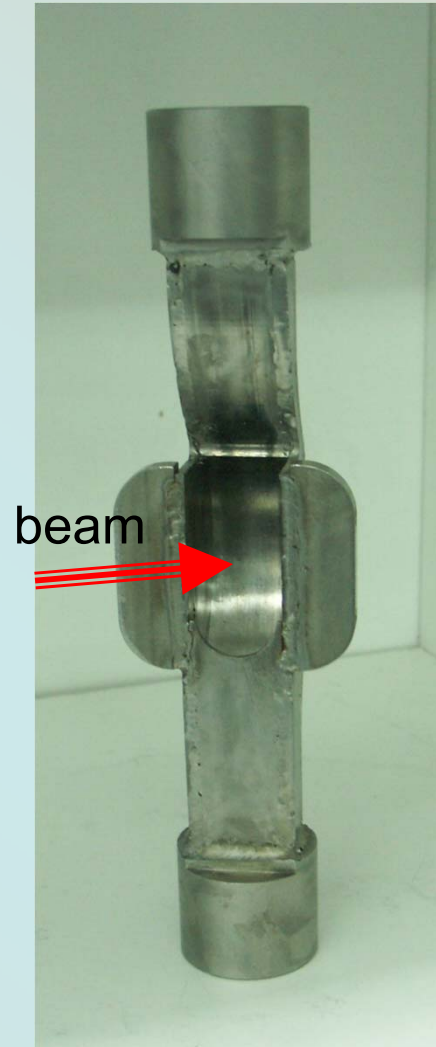
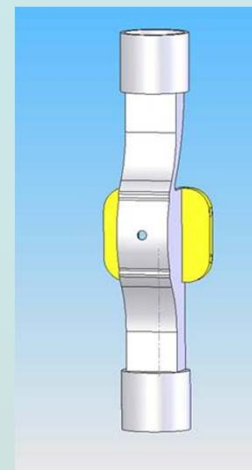
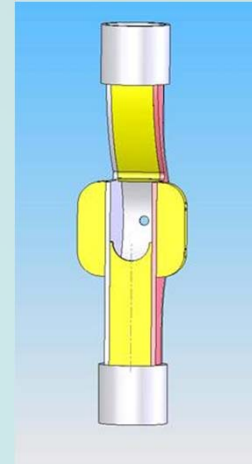
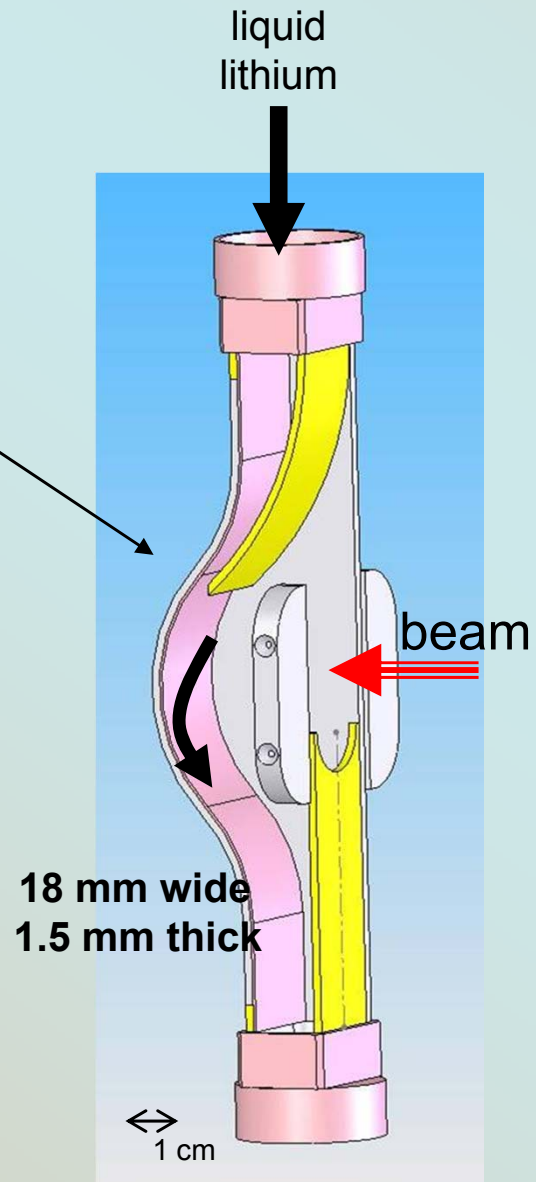
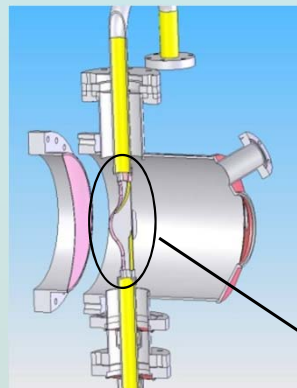
Lithium
containment
tank,
heat
exchanger
and Be-7 cold
trap

Accelerator port

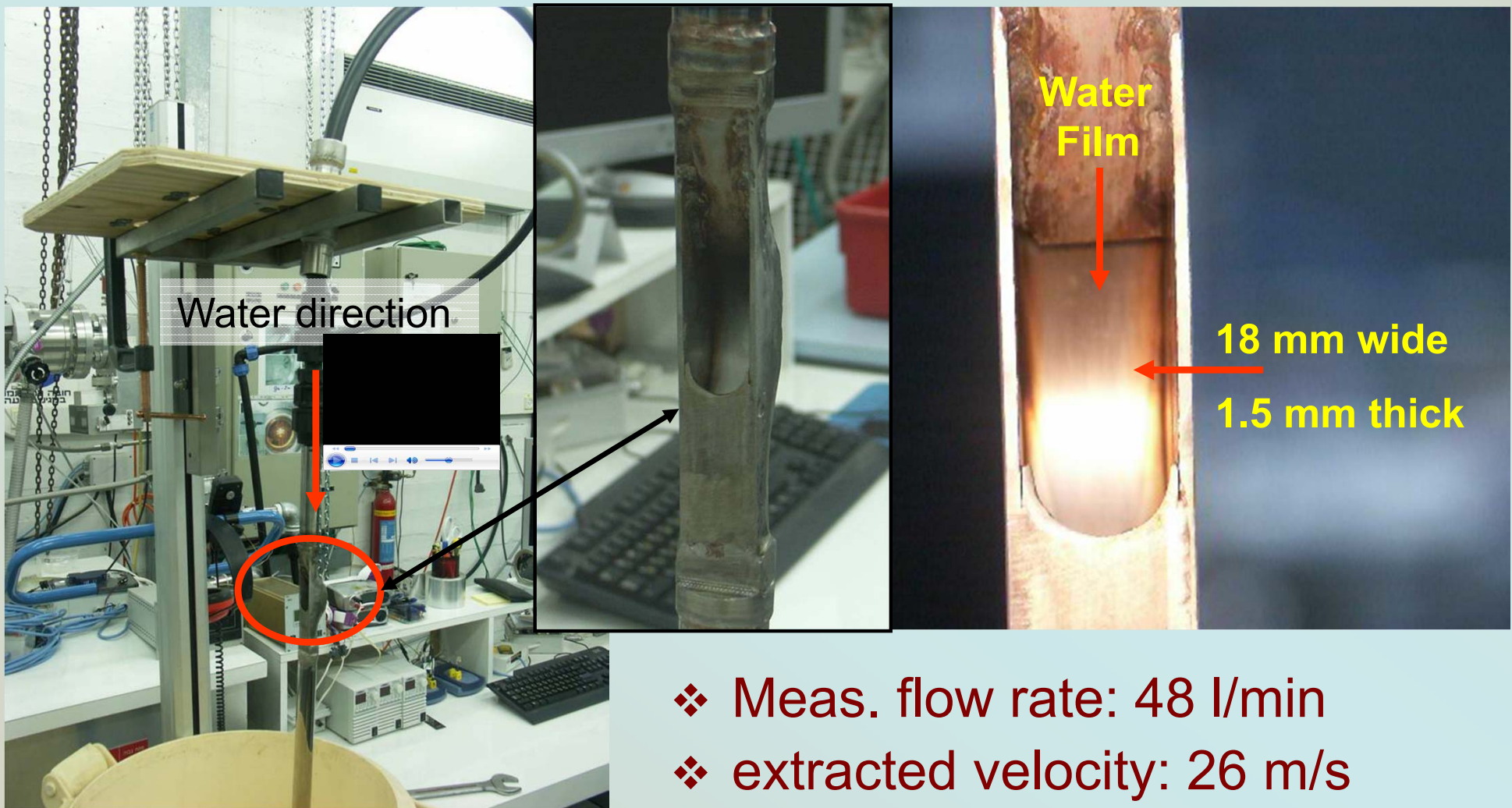
Target chamber



Lithium Nozzle

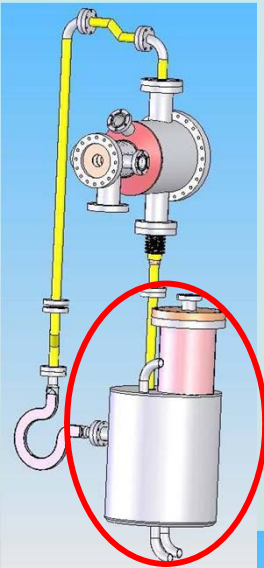


Concave jet - Water test

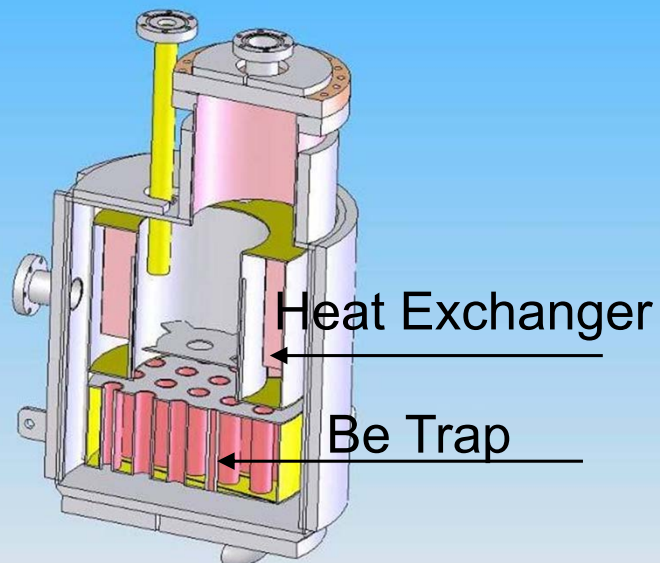


Lithium tank

Design to remove ~12 kW

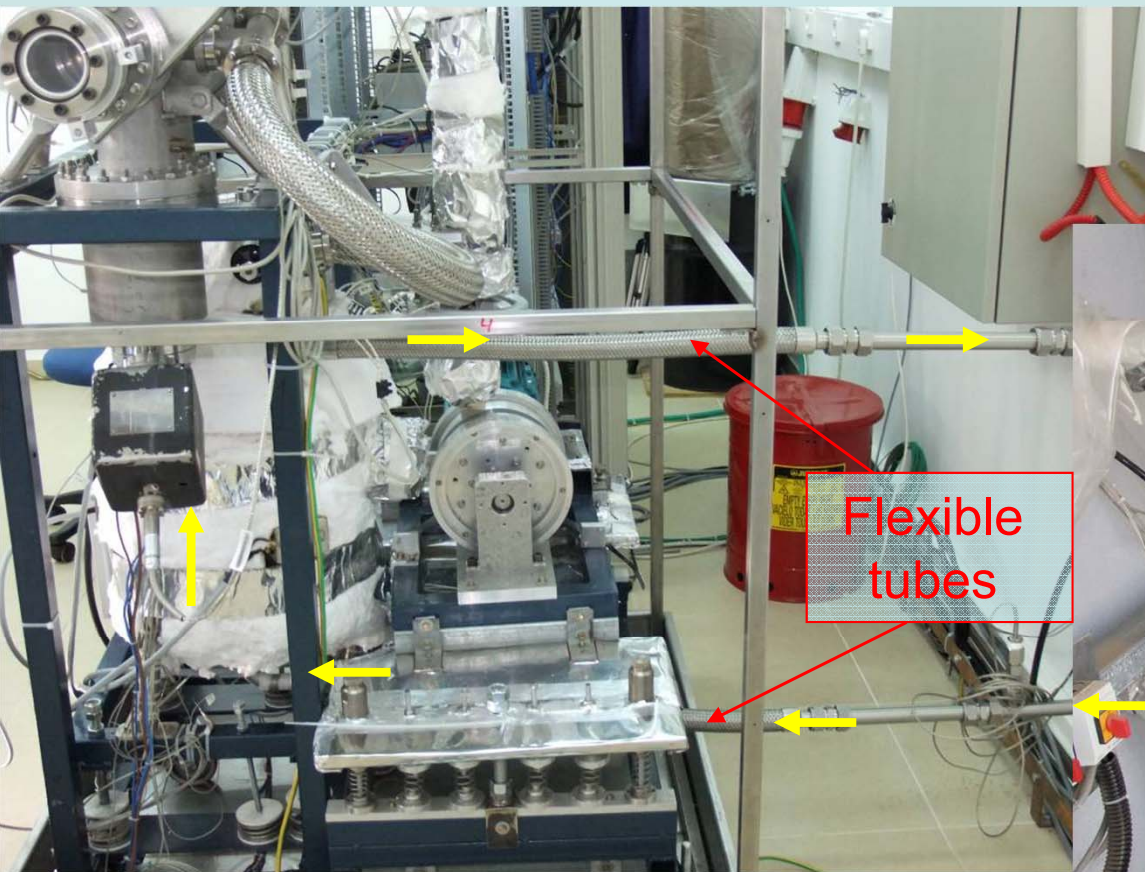


Cross Section

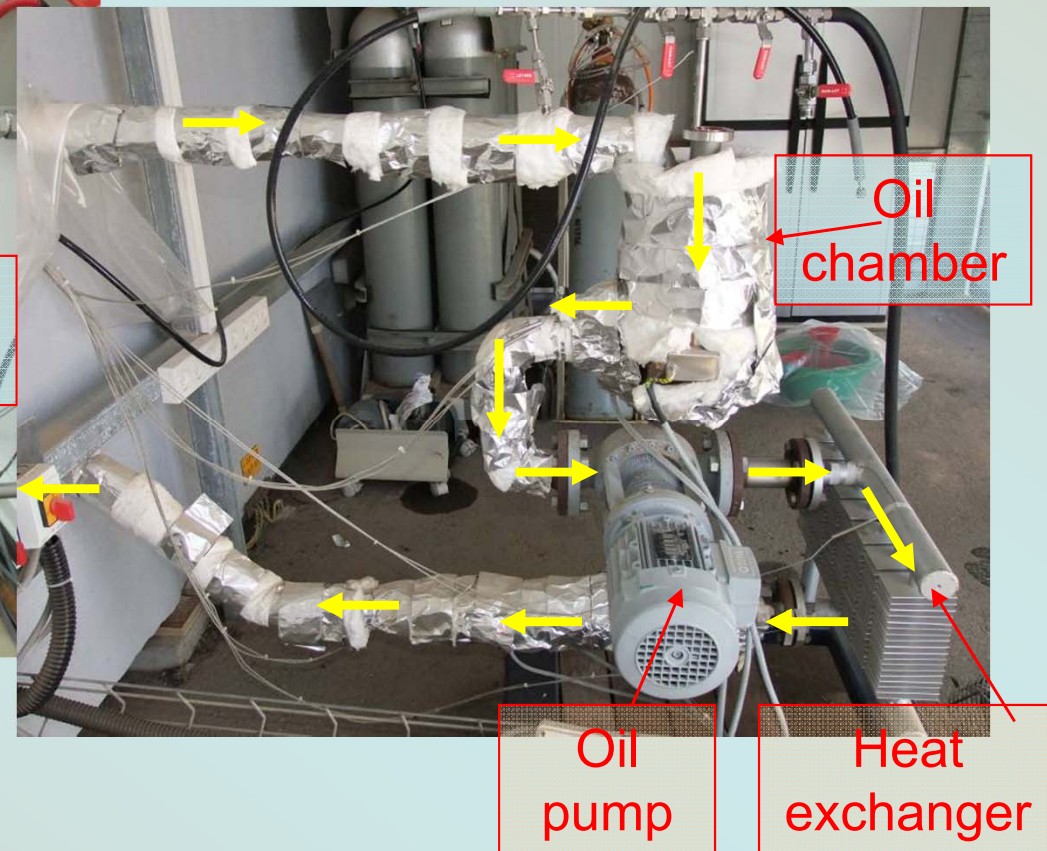


Oil cycle

Inside the lab



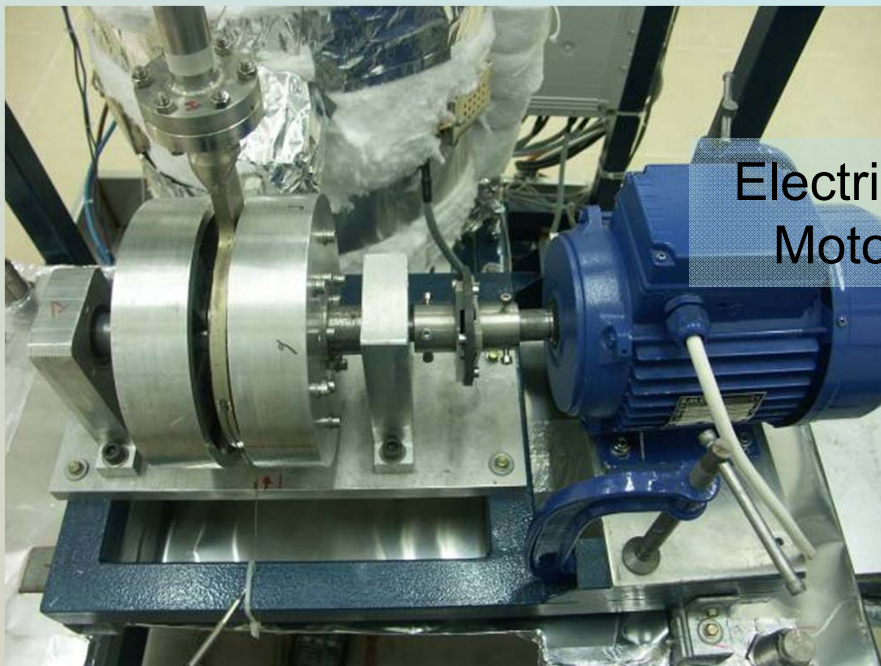
Outside the lab



Electro-magnetic pump



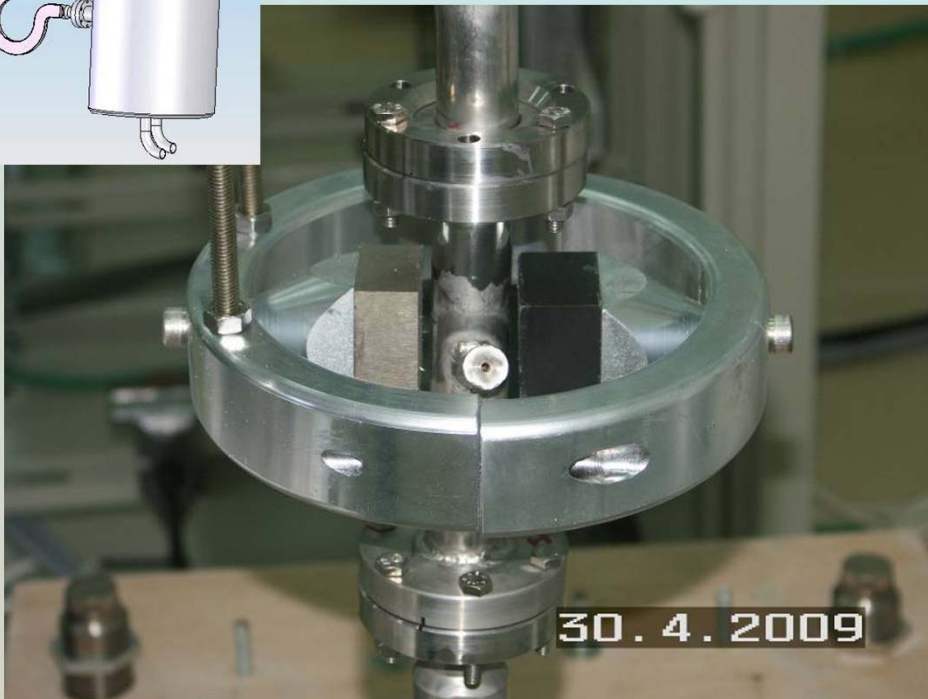
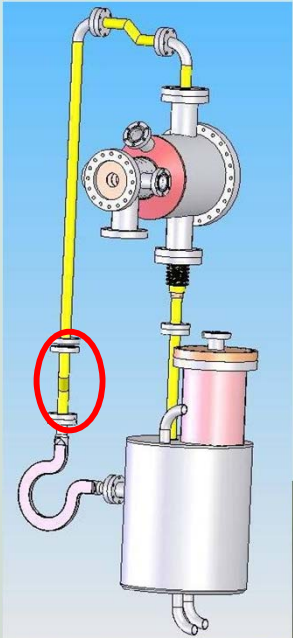
Permanent
SmCo
Magnets



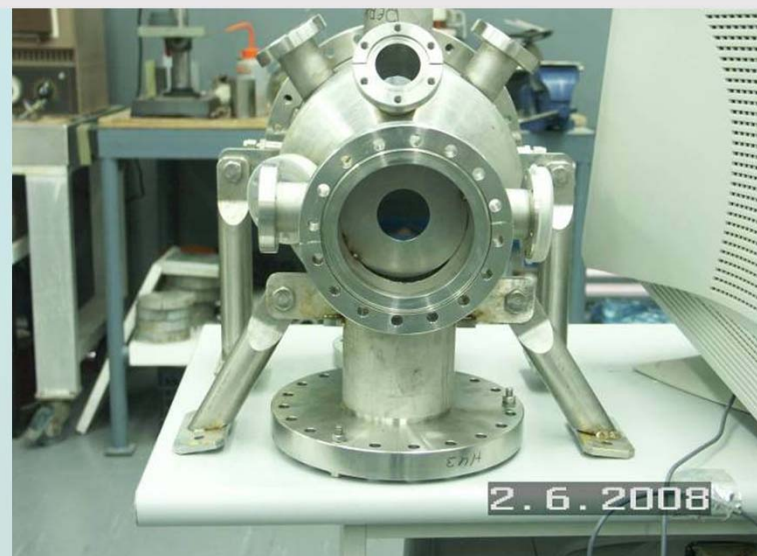
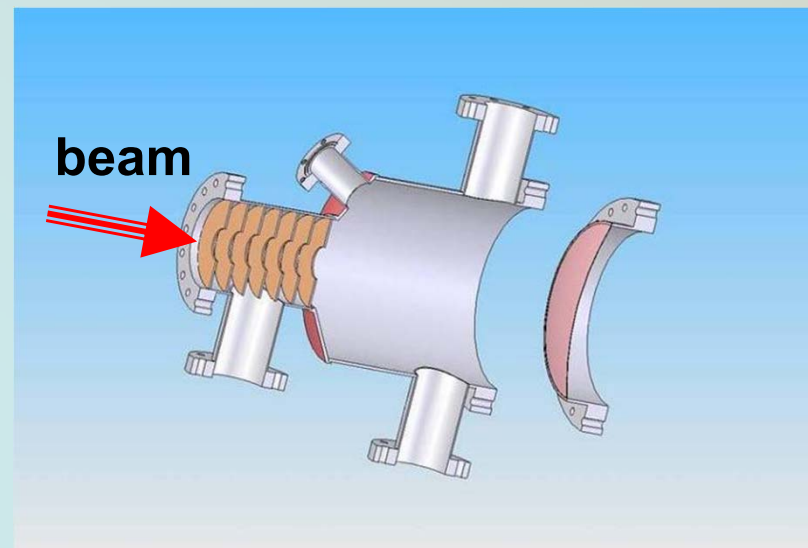
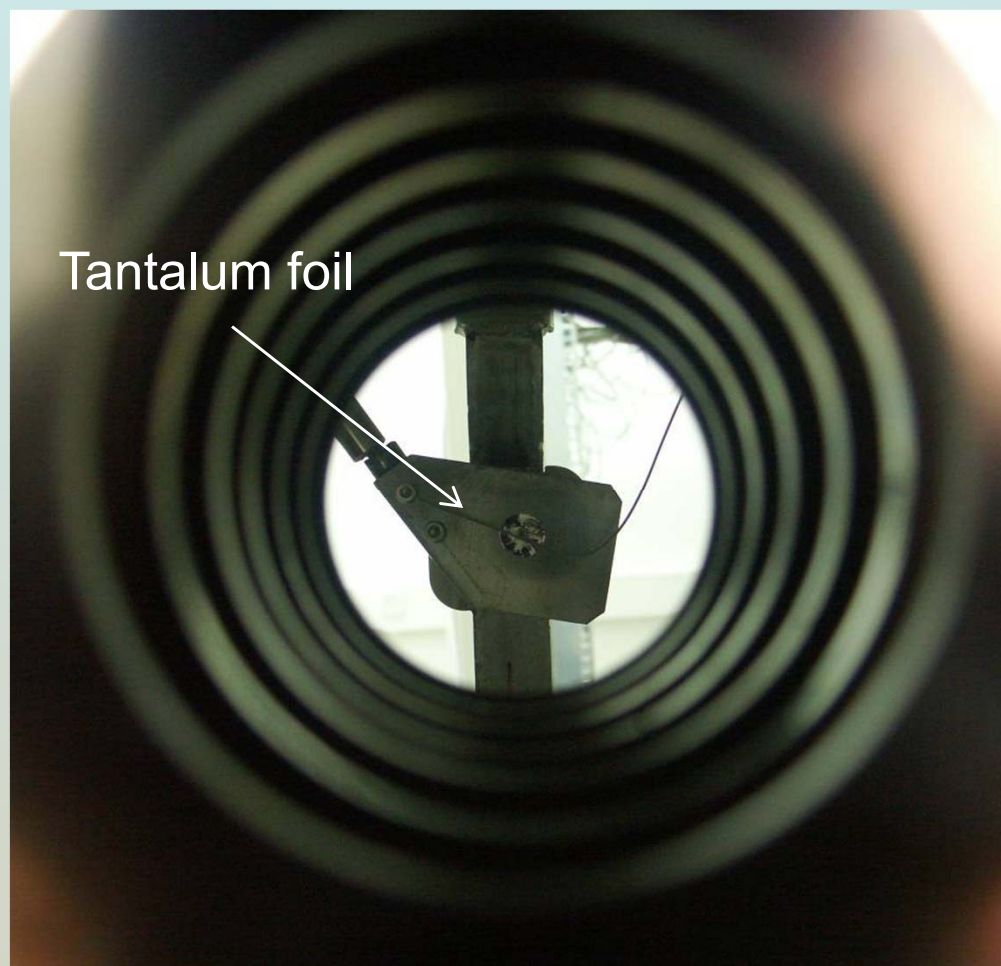
Electrical
Motor



DC electro-magnetic flow meter

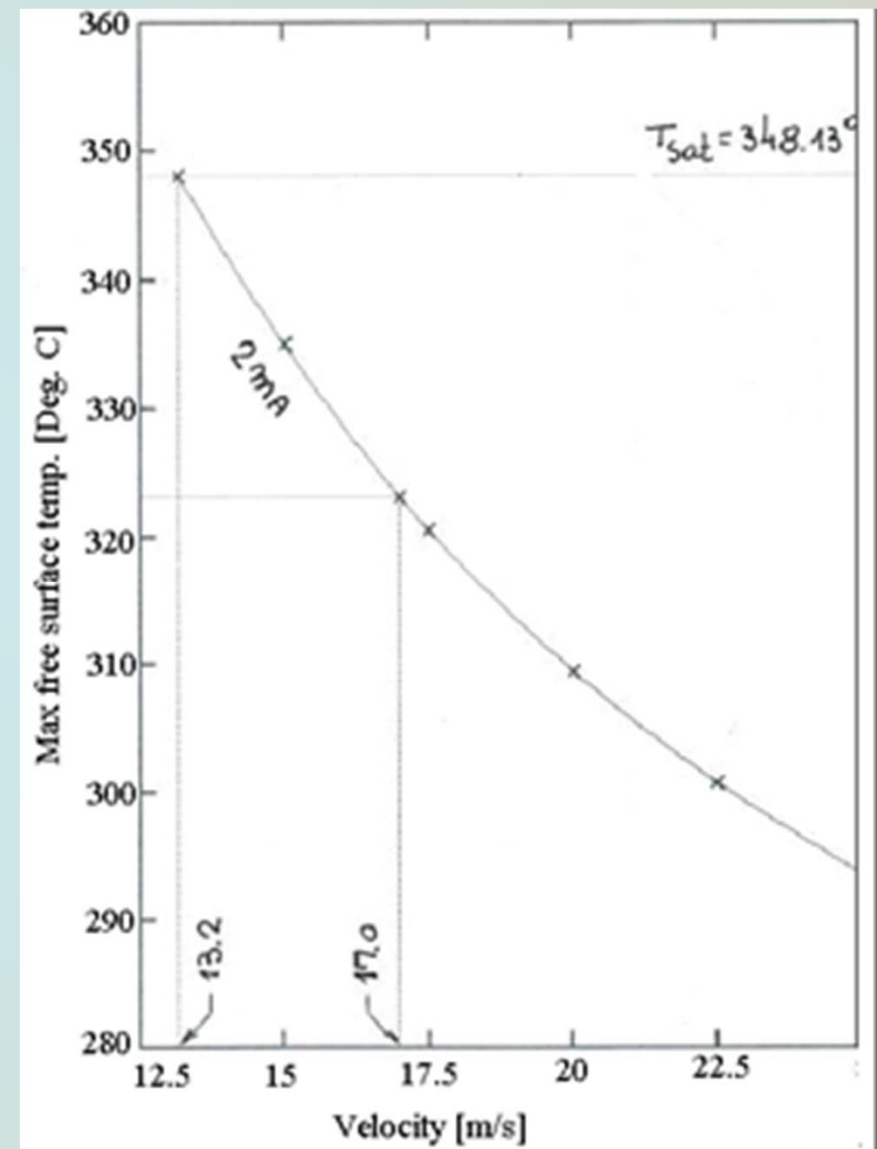
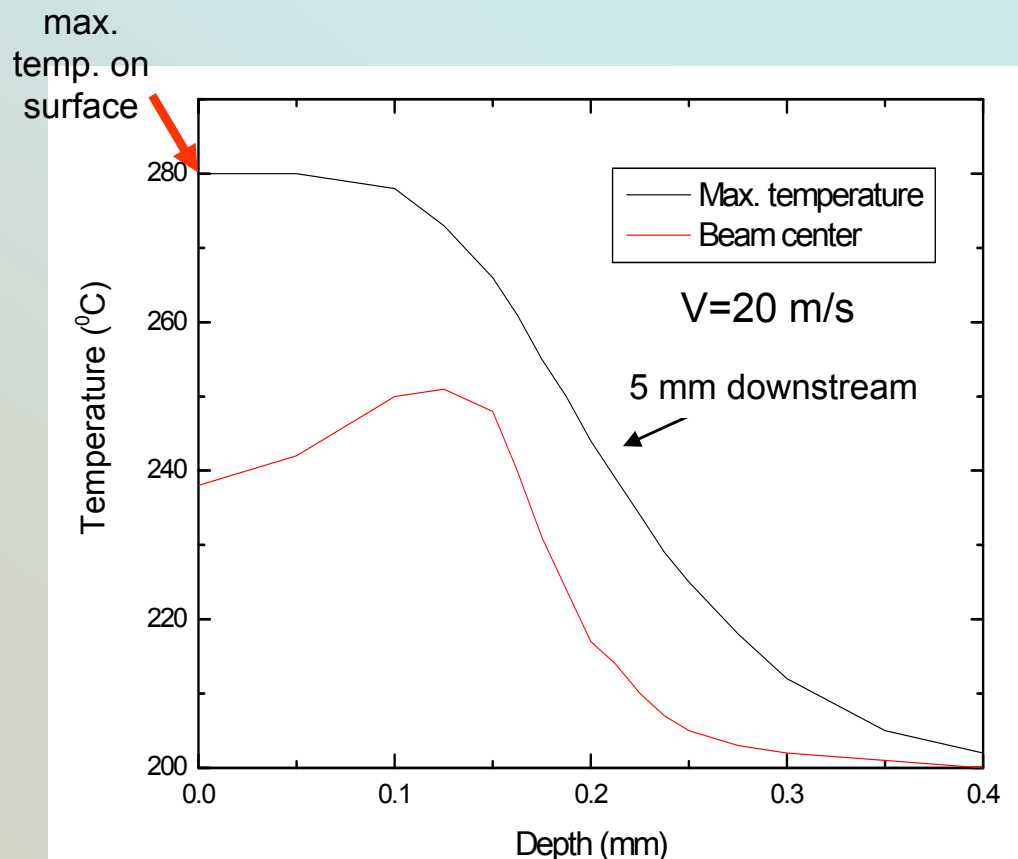


Lithium vapor trap



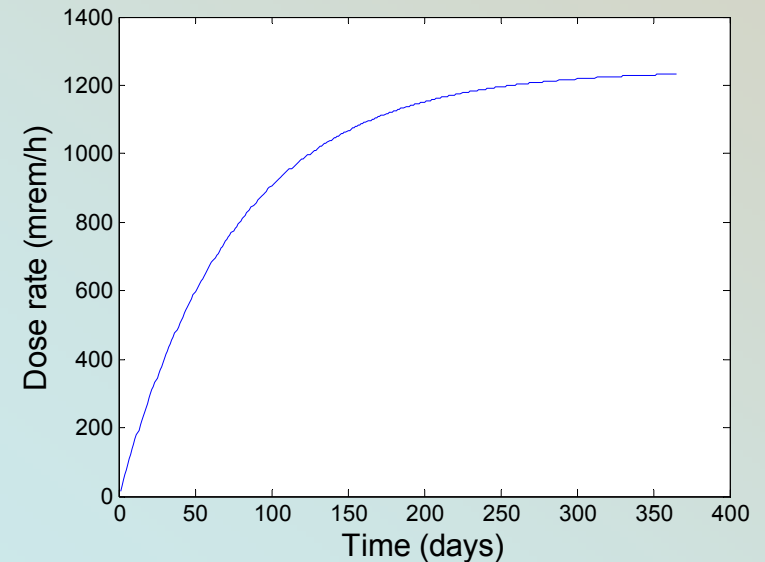
Thermal evaluations

- ❖ Peak temperature elevation at the beam bombarding area
 - Conservative saturation point: 350°C (lithium boiling point at 10^{-5} Torr)



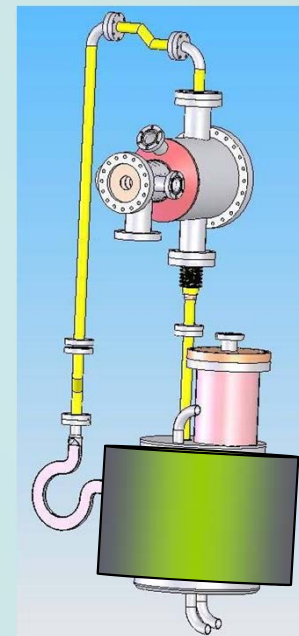
Be-7 production

- ❖ ^7Be : half-life of 53 days, 478 keV gamma radiation.
- ❖ Annual irradiation with 4 mA, 2 MeV proton beam, 8 hours a day, will produce the following dose rate, 30 cm from the system.



Solutions:

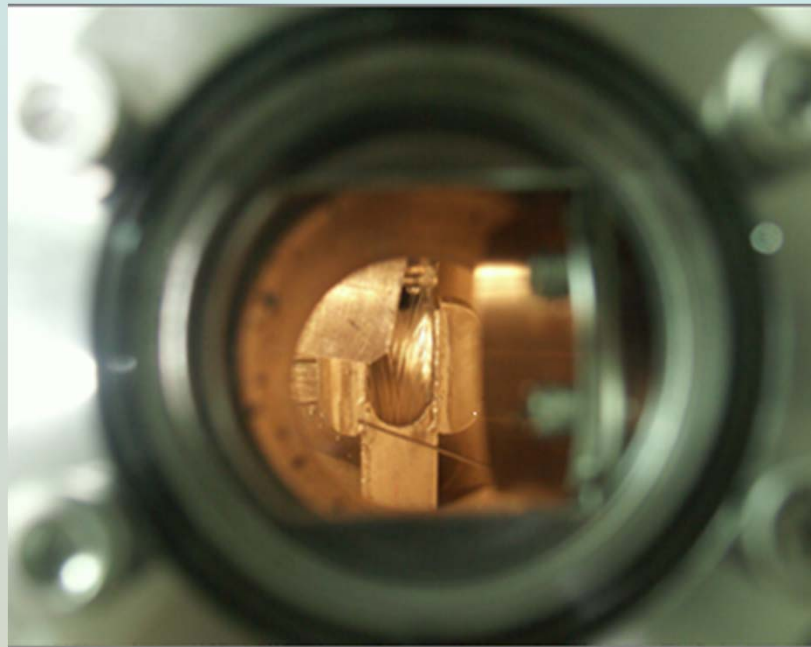
1. Most of the Be-7 will be accumulating at the cold trap and heat exchanger area^[1]. The temperature in the loop and in the cold trap will be set according thermodynamic analysis of ^7Be in molten lithium.
2. The area will be shielded ($\sim 1.5\text{-}3$ cm of Pb).
3. The irradiation periods were calculated in advance in order to control the radiation levels.



^[1] M. Ida et. al., Fusion Engineering and Design 82 (2007) 2490-2496.

Lithium circulation test

- ❖ Lithium heated up to 200°C.
- ❖ Pressure: 8×10^{-6} Torr
- ❖ Velocity: up to 5 m/s
- ❖ Stable and full lithium film

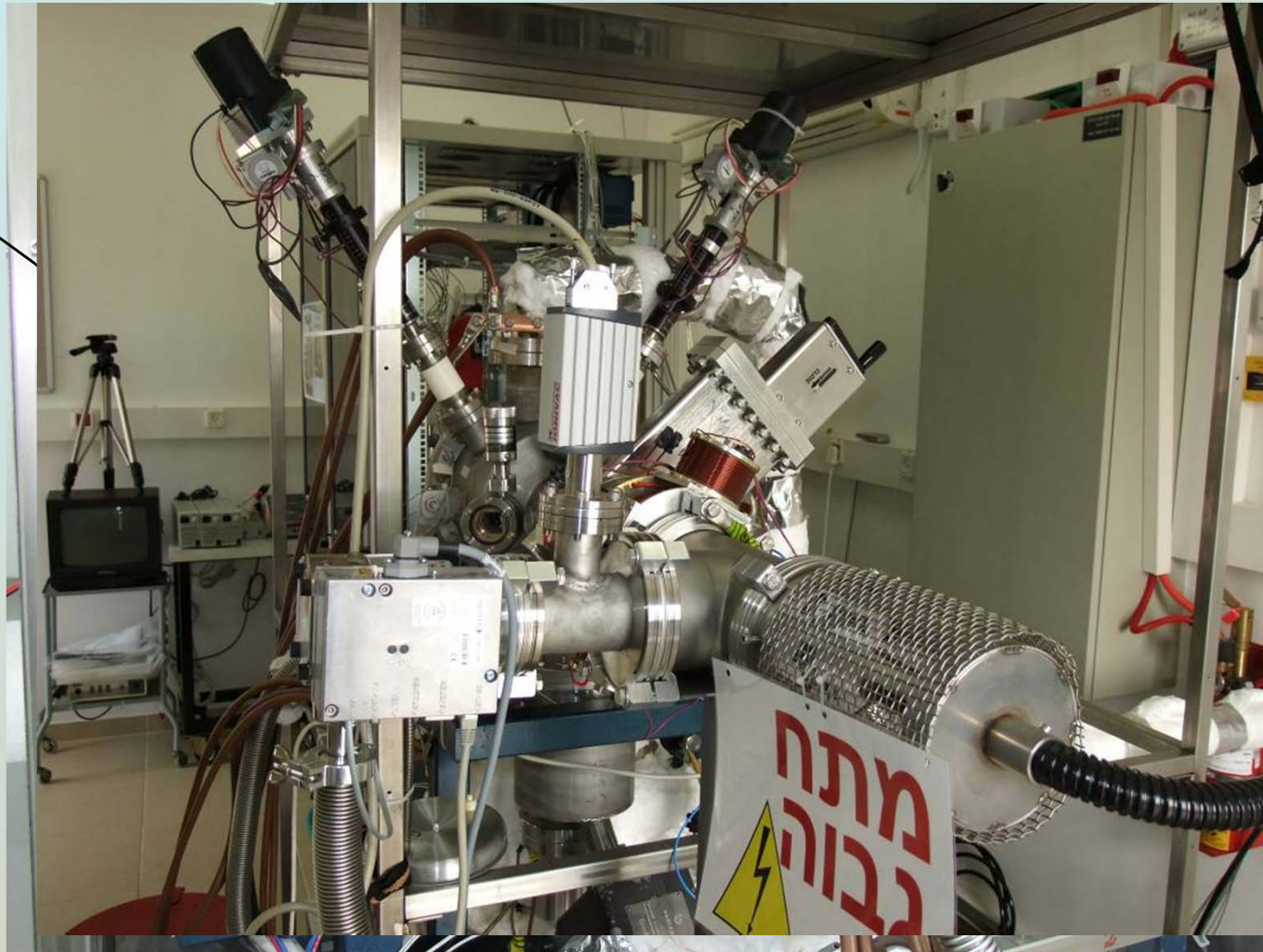


Lithium insertion and circulation movie



1 A, 20 keV (20 kW) electron gun at LiLiT

Beam
dump

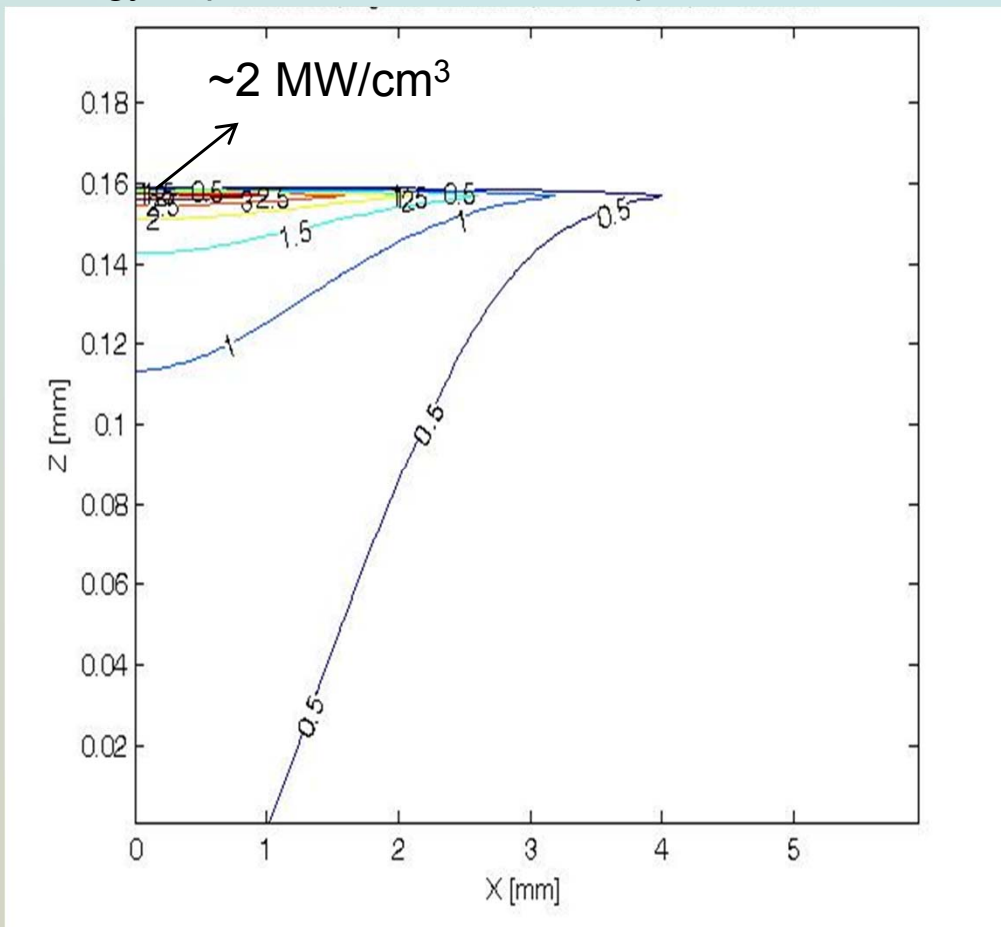


Magnetic
lens

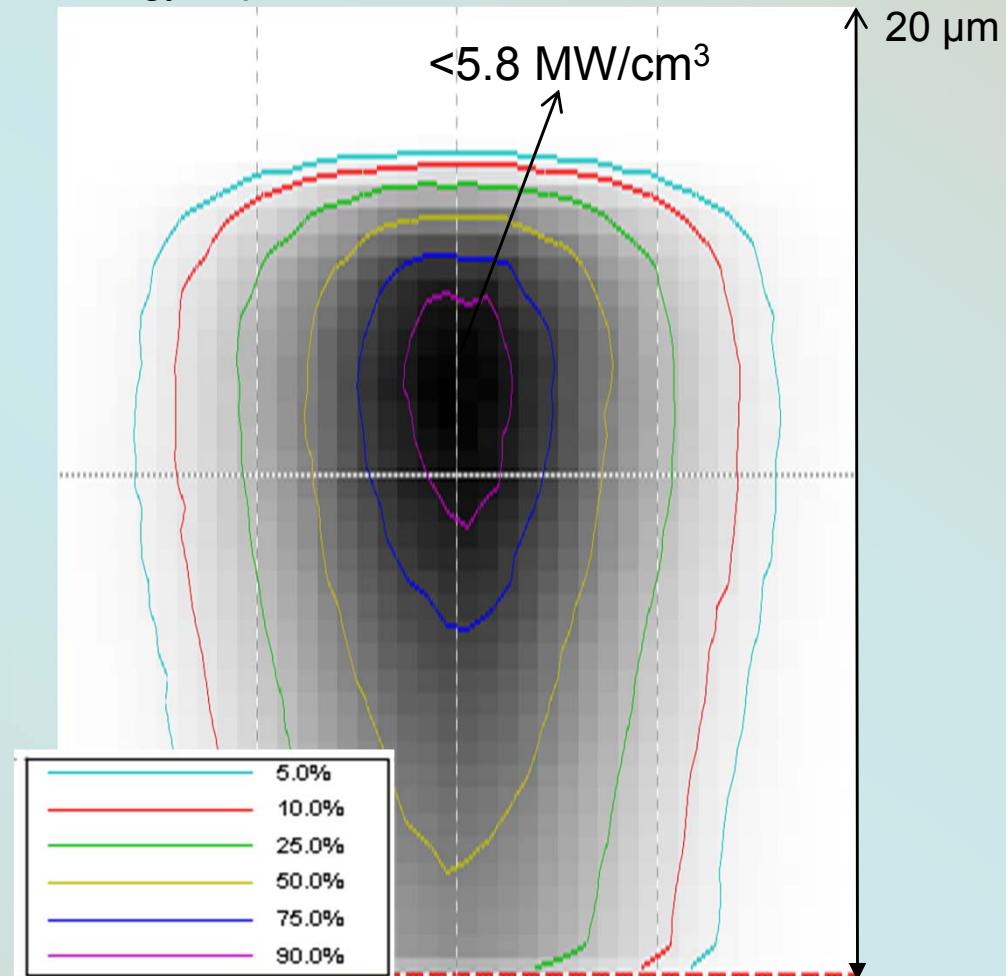
Electron gun off line tests

- ❖ E-gun simulation: High intensity – 20 keV, ~1 A electron gun will simulate thermal deposition of SARAF proton beam.
- ❖ E-gun power density: 5.8 MW/cm³ at 1 A

energy deposition of 2 MeV, 2mA protons in lithium

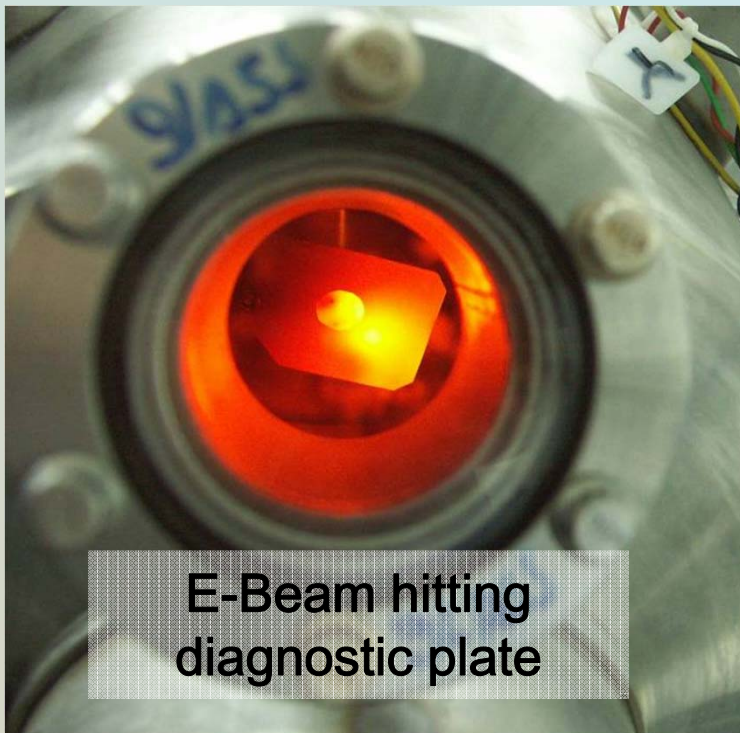


energy deposition of 20 keV electrons in lithium

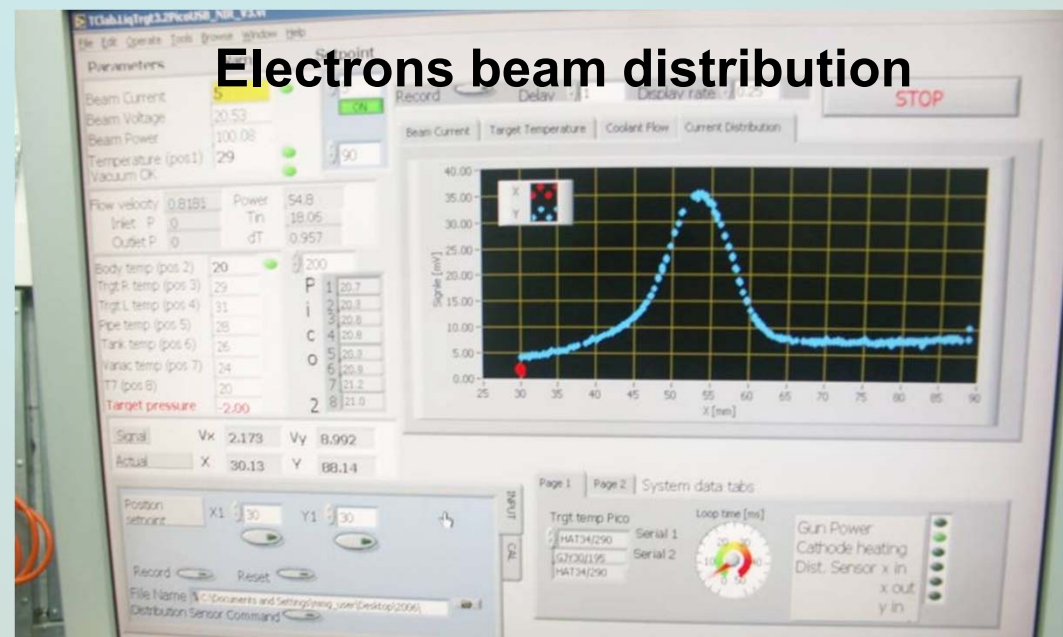


E-gun experiment

- ❖ E-beam focusing, using magnetic lens, on diagnostic plate
- ❖ Measurement of e-beam distribution (up to 10 mA)
- ❖ Applying higher beam power on the lithium flow



E-Beam hitting diagnostic plate



e-gun experiment results

- ❖ Electron Beam shape measurement
- ❖ Velocity measurement - ~ 3 m/s (~ 30 % of EM pump capability)
- ❖ Stable lithium flow at irradiation up to 2 kW (at 3 m/s)
- ❖ Excessive evaporation when ~ 2.2 kW beam was applied (at 3 m/s)

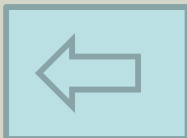
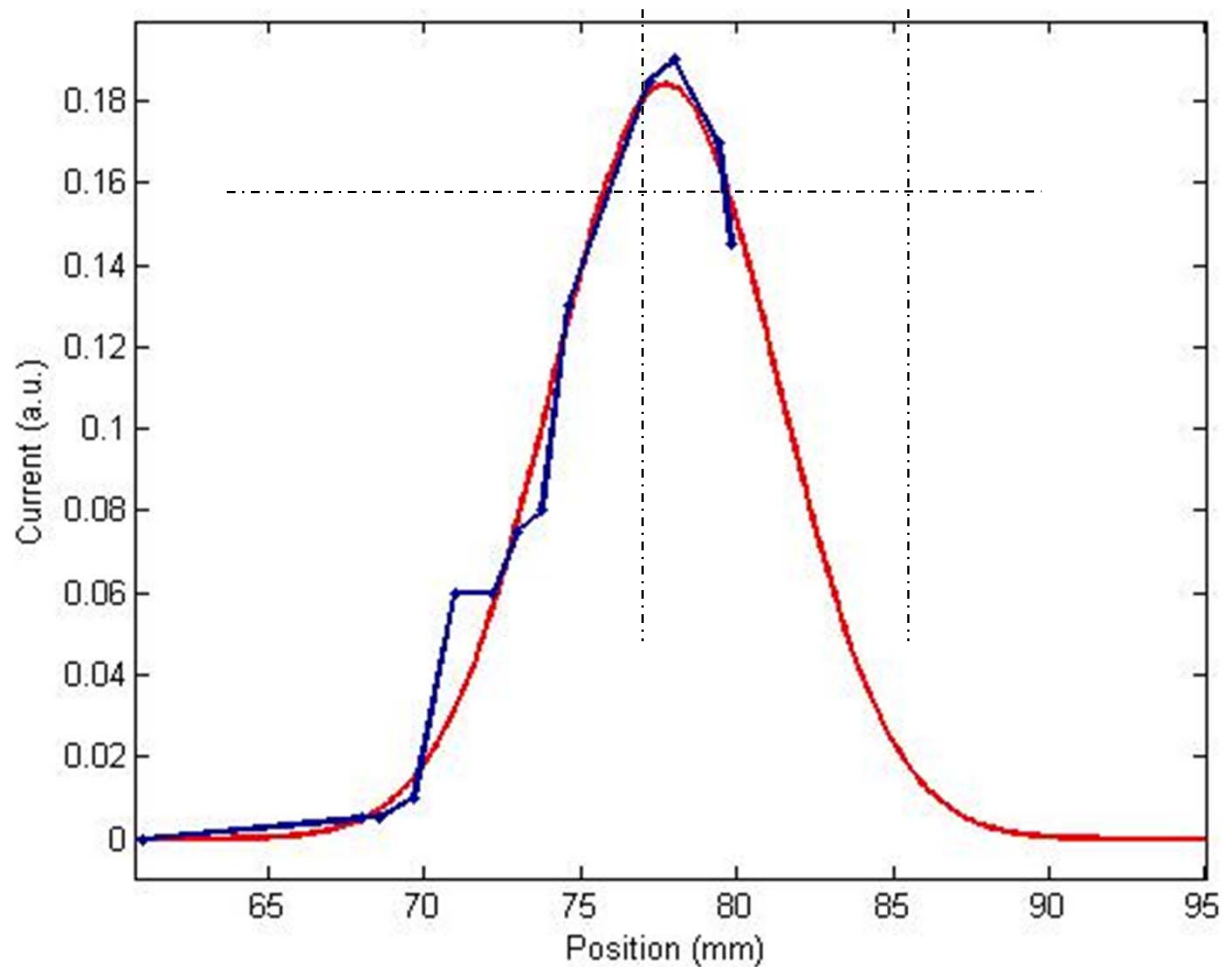


E- beam profile:

Blue: measurement of 10 mA beam

Red: Gaussian with FWHM= 8.3 mm and $\sigma=3.5$ mm

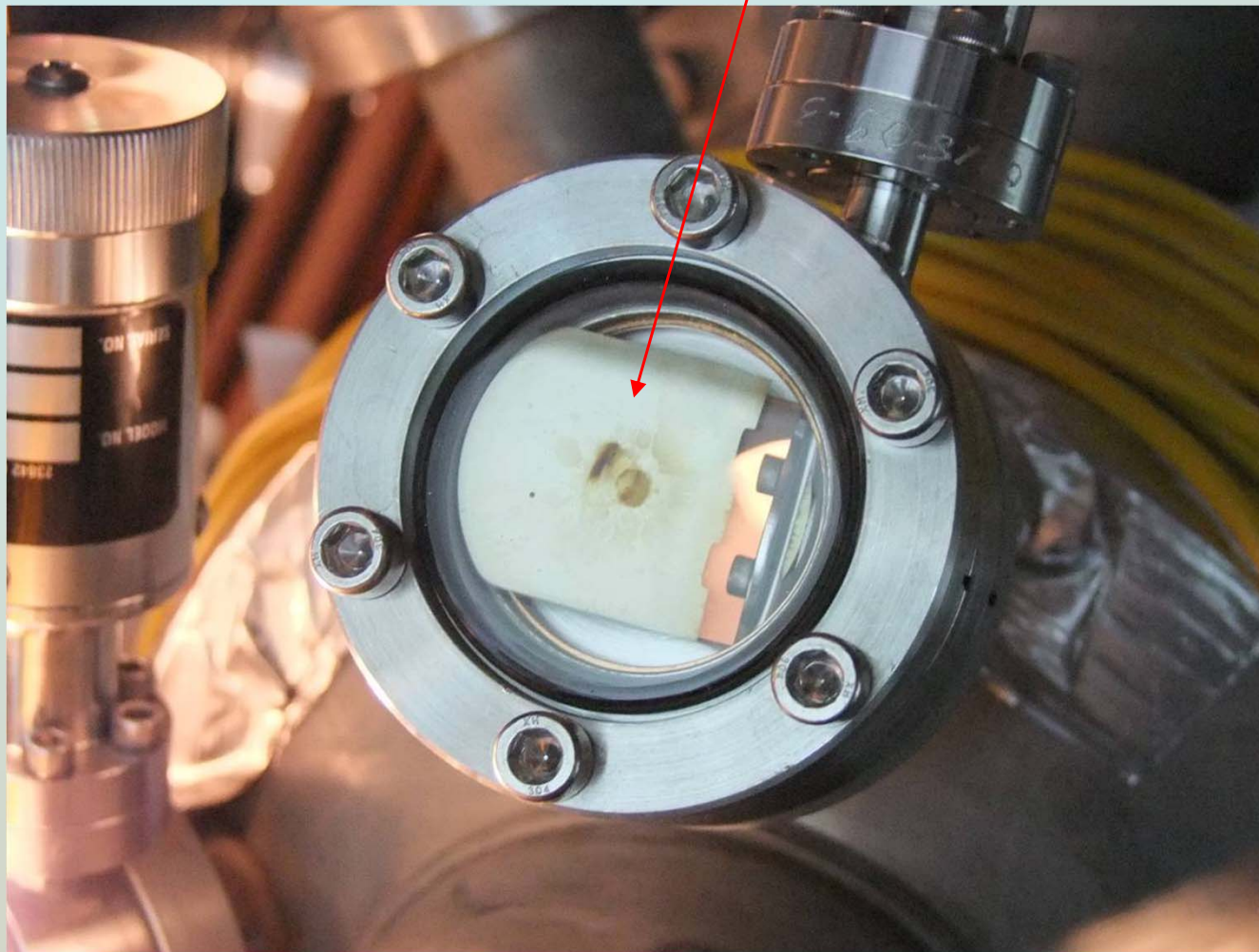
**The maximal electron beam power density was 2.85 kW/cm²
(0.83 MW/cm³)**



e-gun on lithium

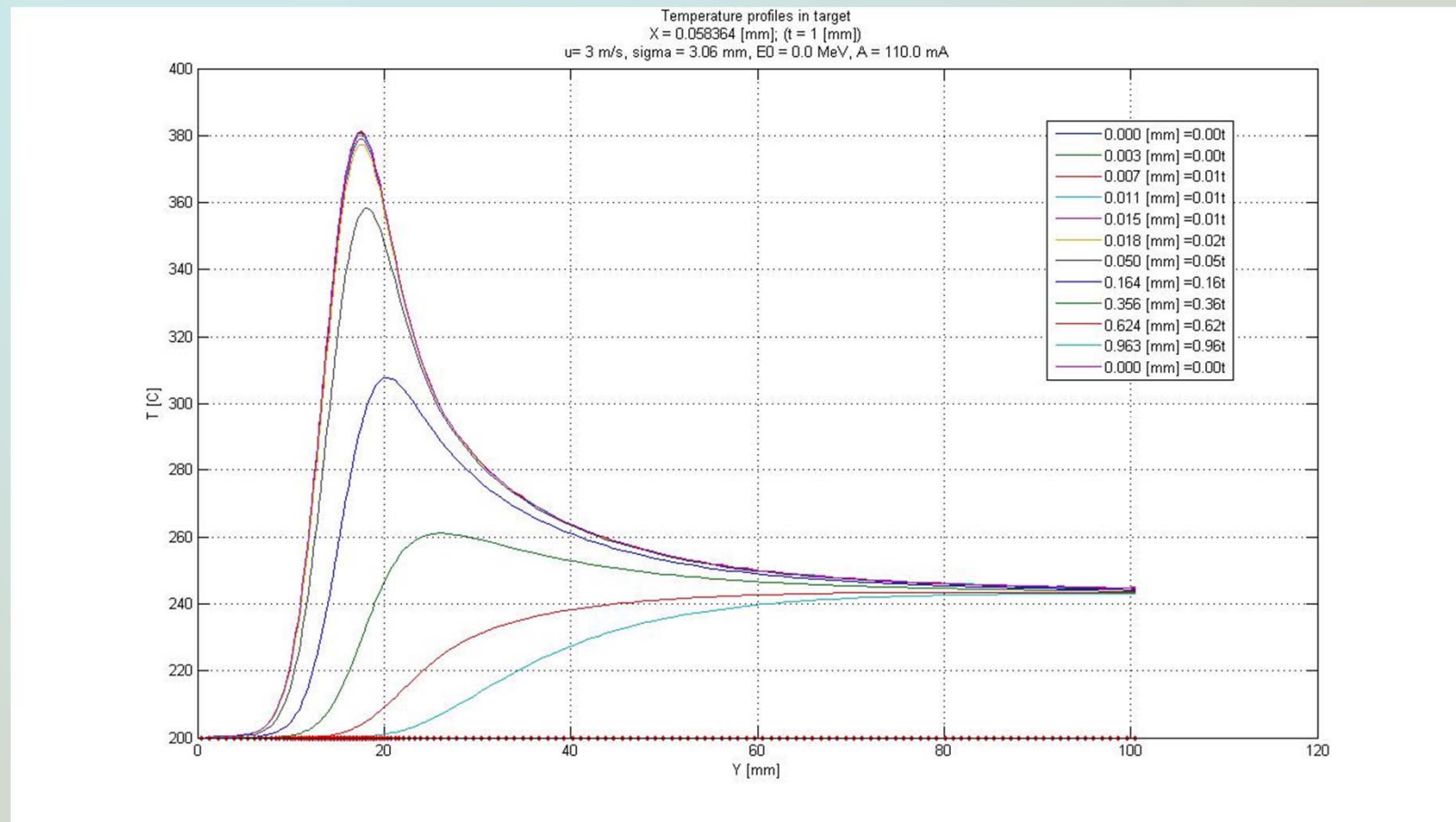


Lithium vapors on viewport window



Temperature calculation for 2.2 kW electron irradiation

Calculated Max temperature = **380°C**
 Expected saturation temperature: **350°C**



Flow →

Our future plans

- ❖ E-gun irradiation at higher flow velocity
- ❖ Transportation and connection to SARAF accelerator beam line
- ❖ Proton beam heat removal experiments
- ❖ Be-7 dynamics in the system
- ❖ Neutron measurements

The LiLiT Team:

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N. Hazenshprung, D. Kijel, A. Nagler,
I. Silverman**

Thanks to J. Nolen, C. Reed & Y. Momozaki
for the help with design and training

Thank you

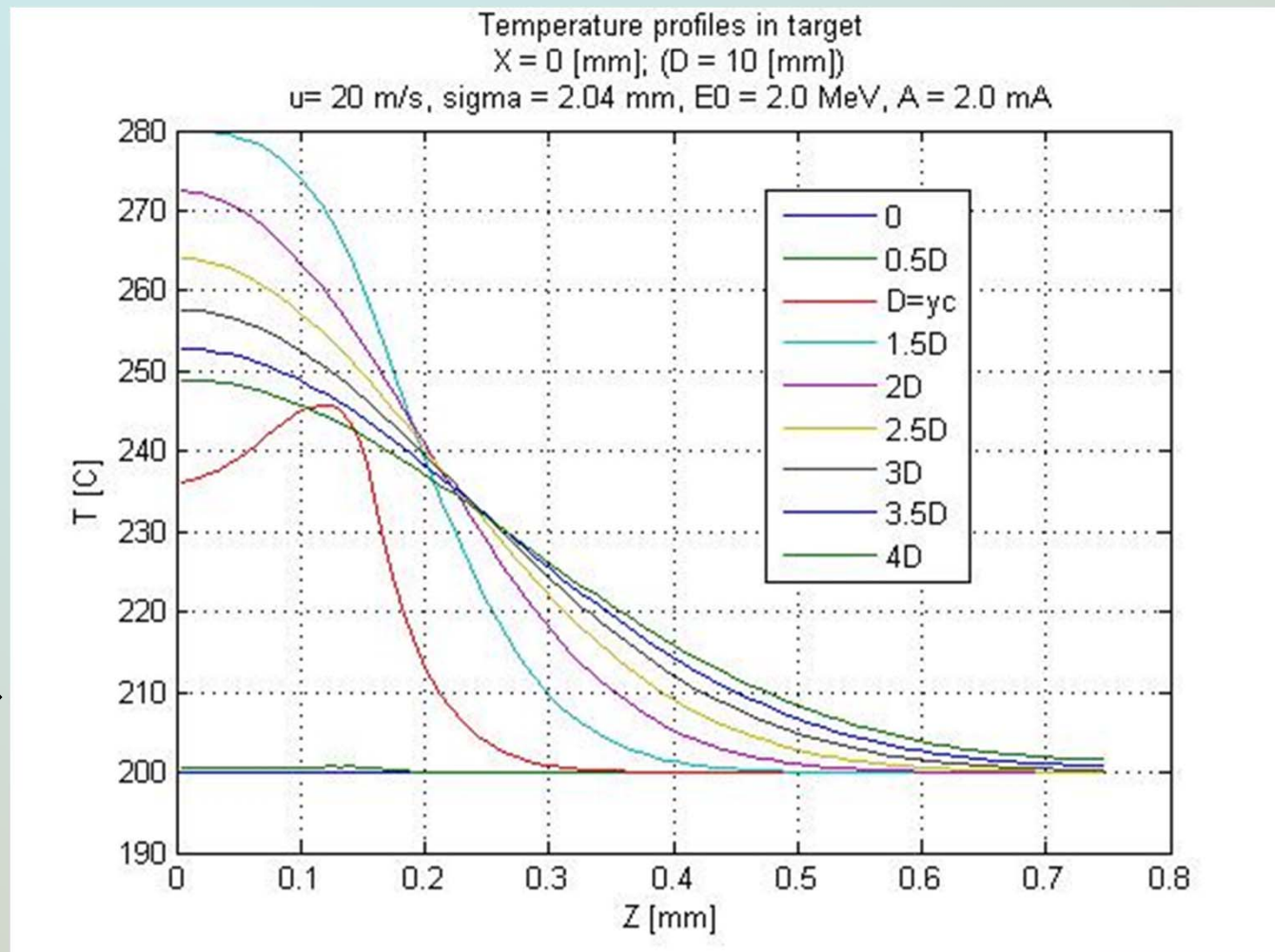
Fire-proof dry room for 20 keV
e-gun experiments



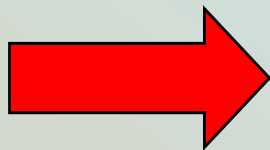
As built system



LiLit @ 4kW heating power



Beam

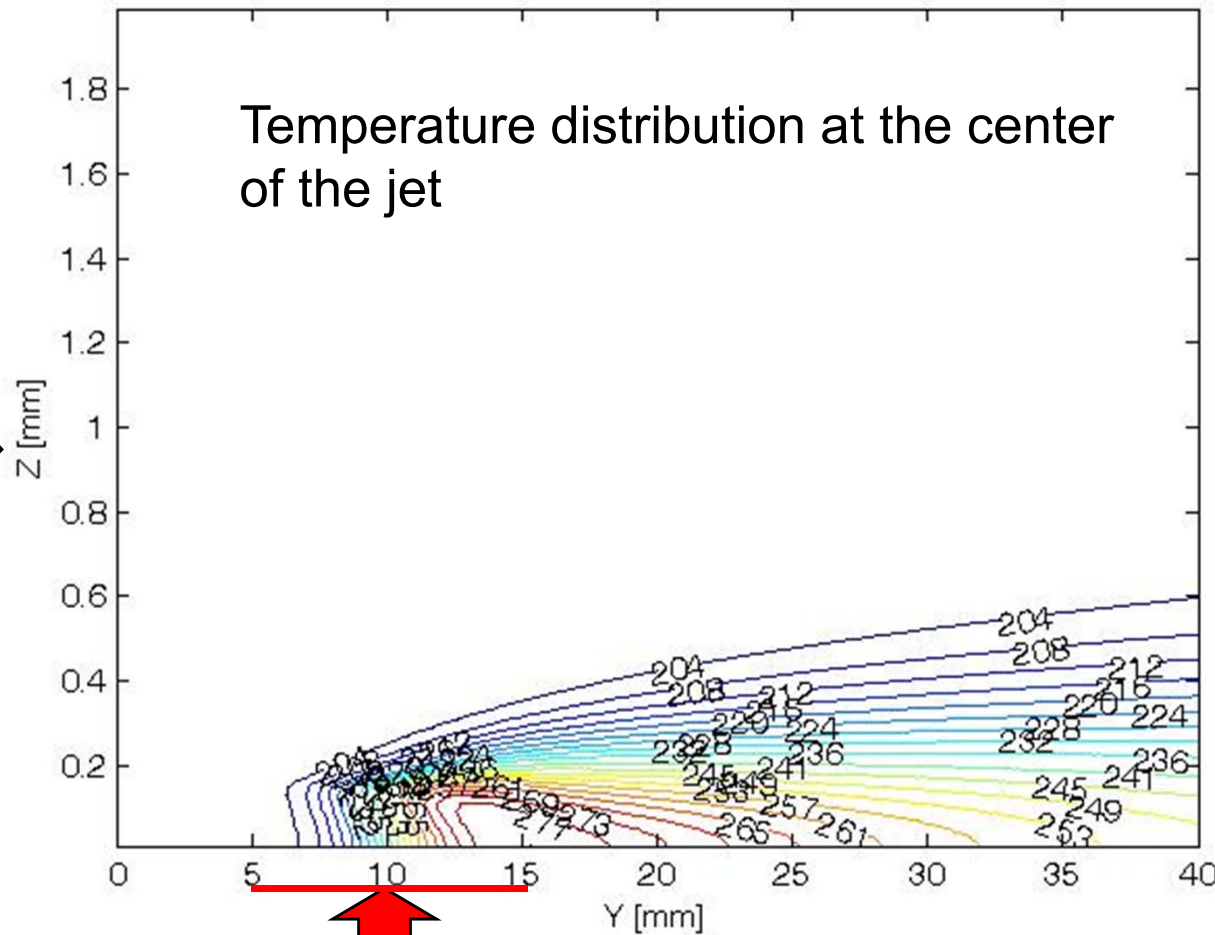


Depth wise temperature distribution

LiLit @ 4kW heating power

Temperature contours
 2D sim, $u = 20$ m/s, $\sigma = 2.04$ mm, $E_0 = 2.0$ MeV, $A = 2.0$ mA

Temperature distribution at the center
 of the jet



Flow
 direction

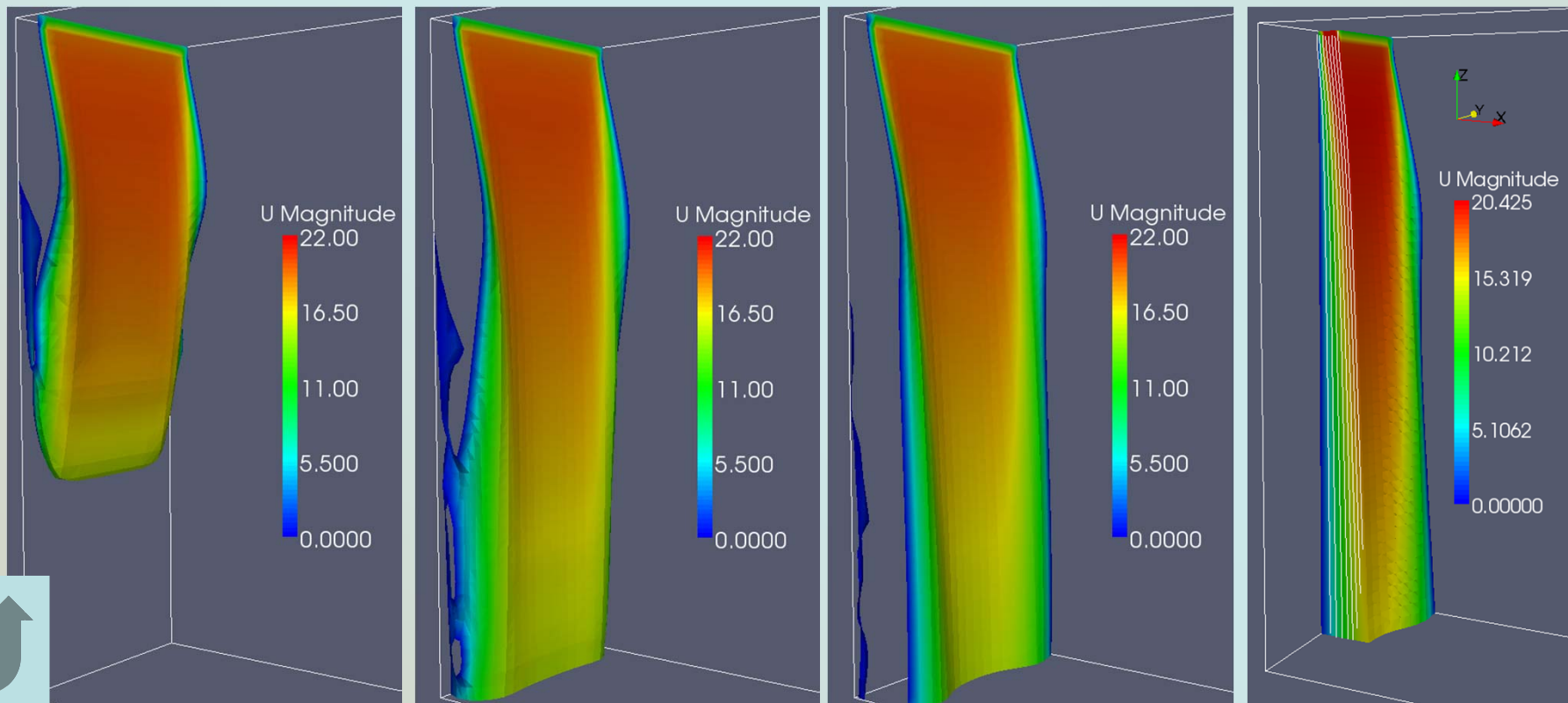


Beam



CFD simulations

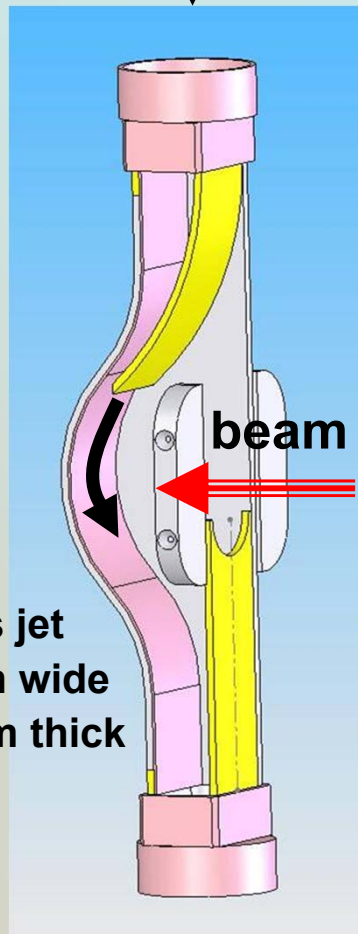
- ❖ 3D flow simulations are done with OpenFoam (open source CFD code)
- ❖ Currently only strait wall jet flow is simulated
- ❖ Planed improvements include concave flow and power deposition



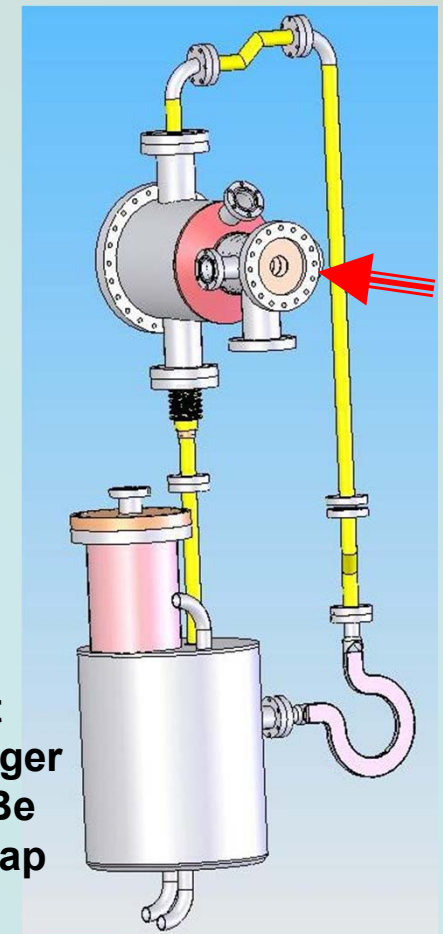
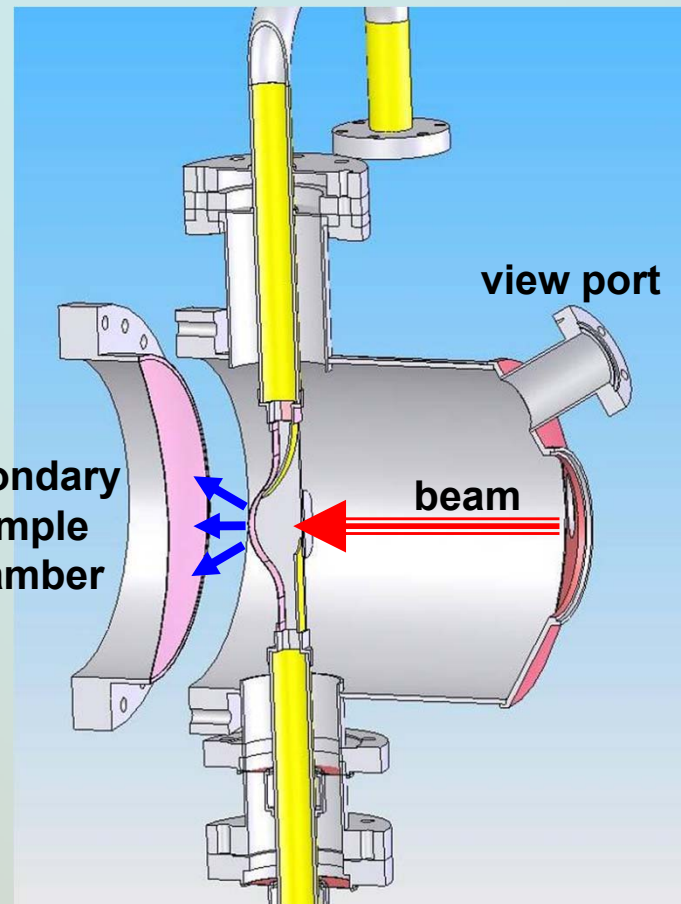
LiLiT jet chamber

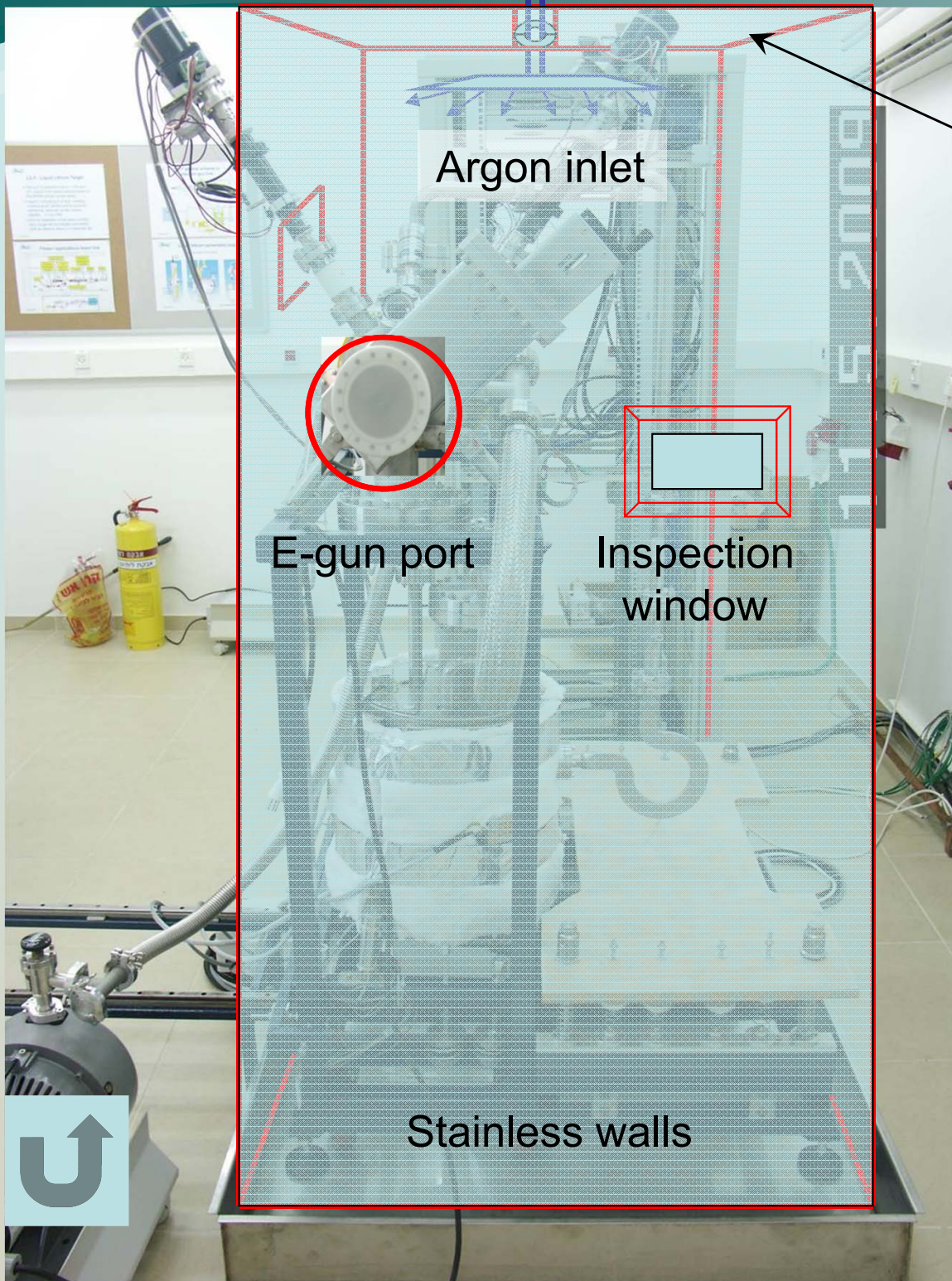
- ❖ built for 2 MeV 3.5 mA protons
- ❖ Gaussian beam spot size with $\sigma=2$ mm

liquid lithium
↓



20 m/s jet
18 mm wide
1.5 mm thick



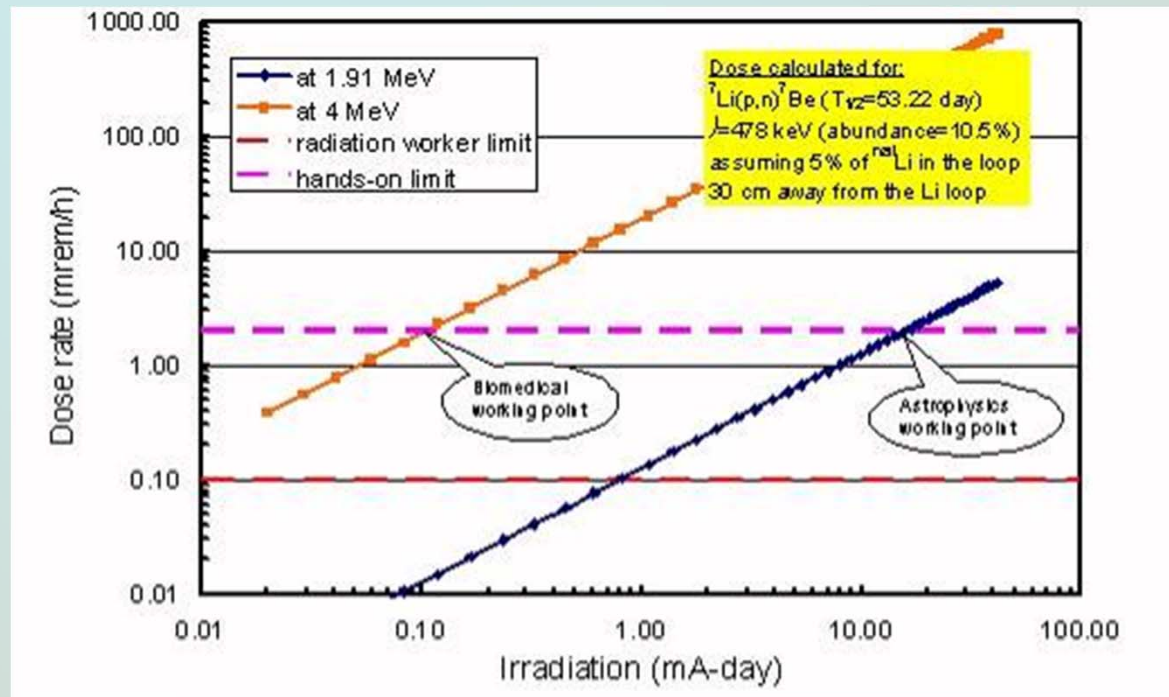


Explosion roof,
held on hinges

Stainless steel fire protection enclosure

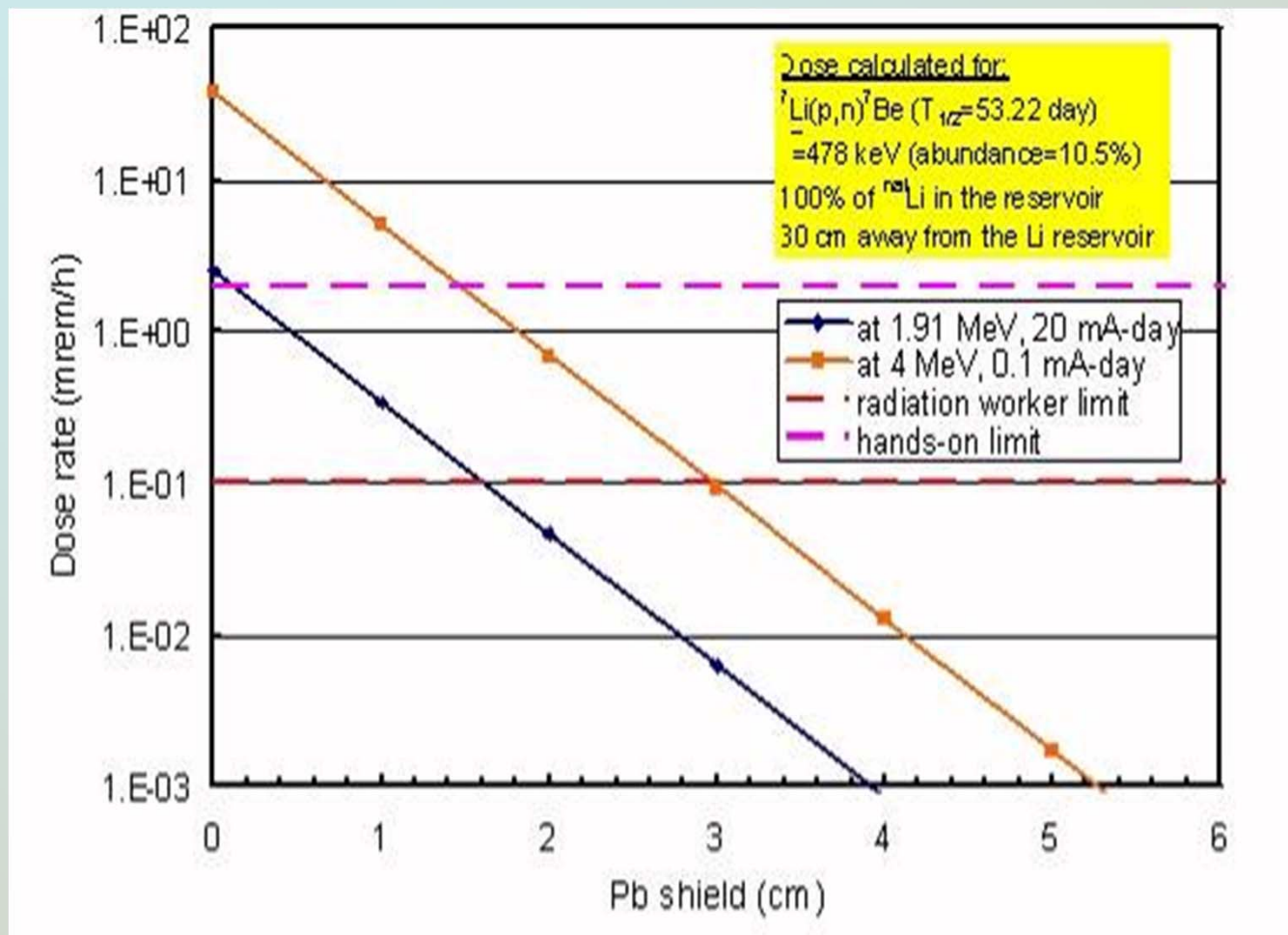


Radiation from ^7Be



The LiLiT loop dose rate as function of integral irradiation duration and intensity. Based on the assumption that 5% of the Li is left in the loop

Radiation shielding



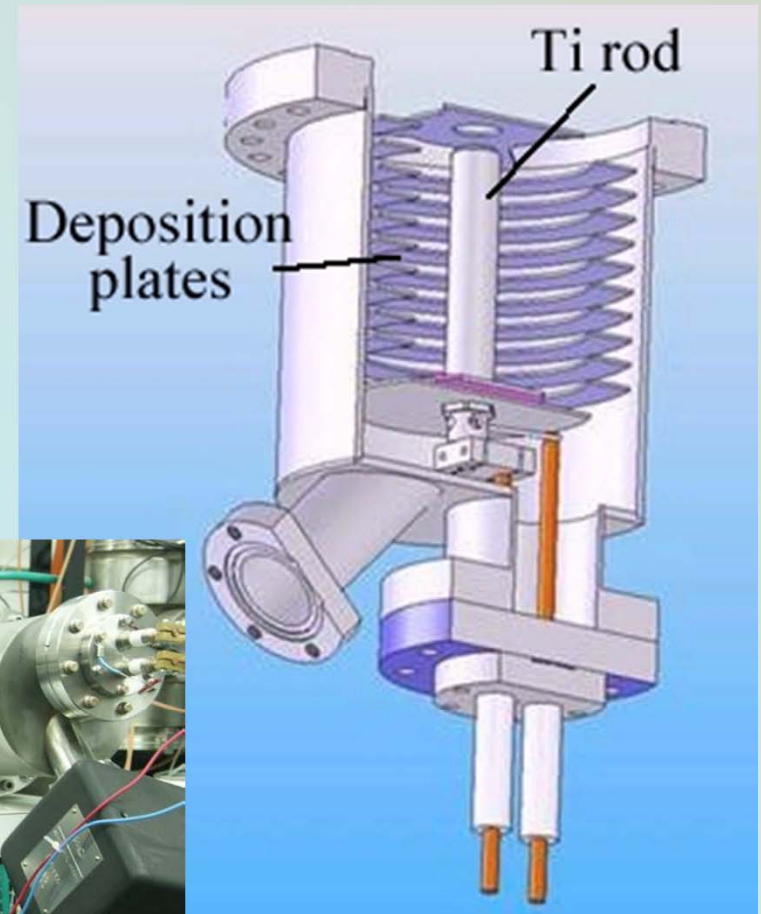
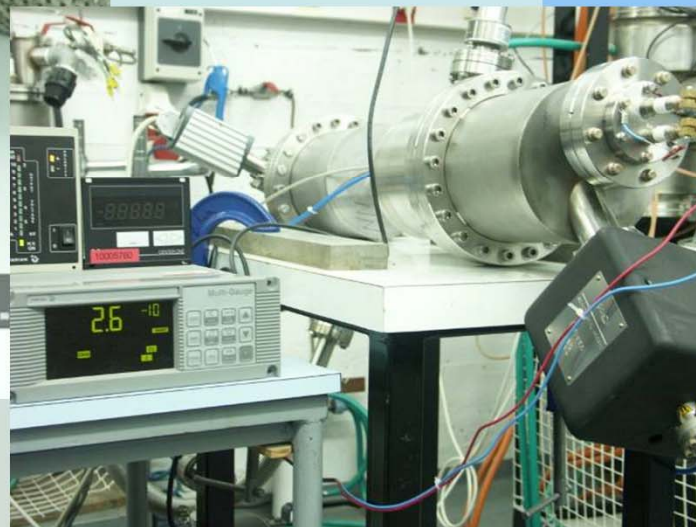
Li reservoir dose rate 30 cm behind a lead shield as function of the lead thickness

Electro-magnetic pump parameters

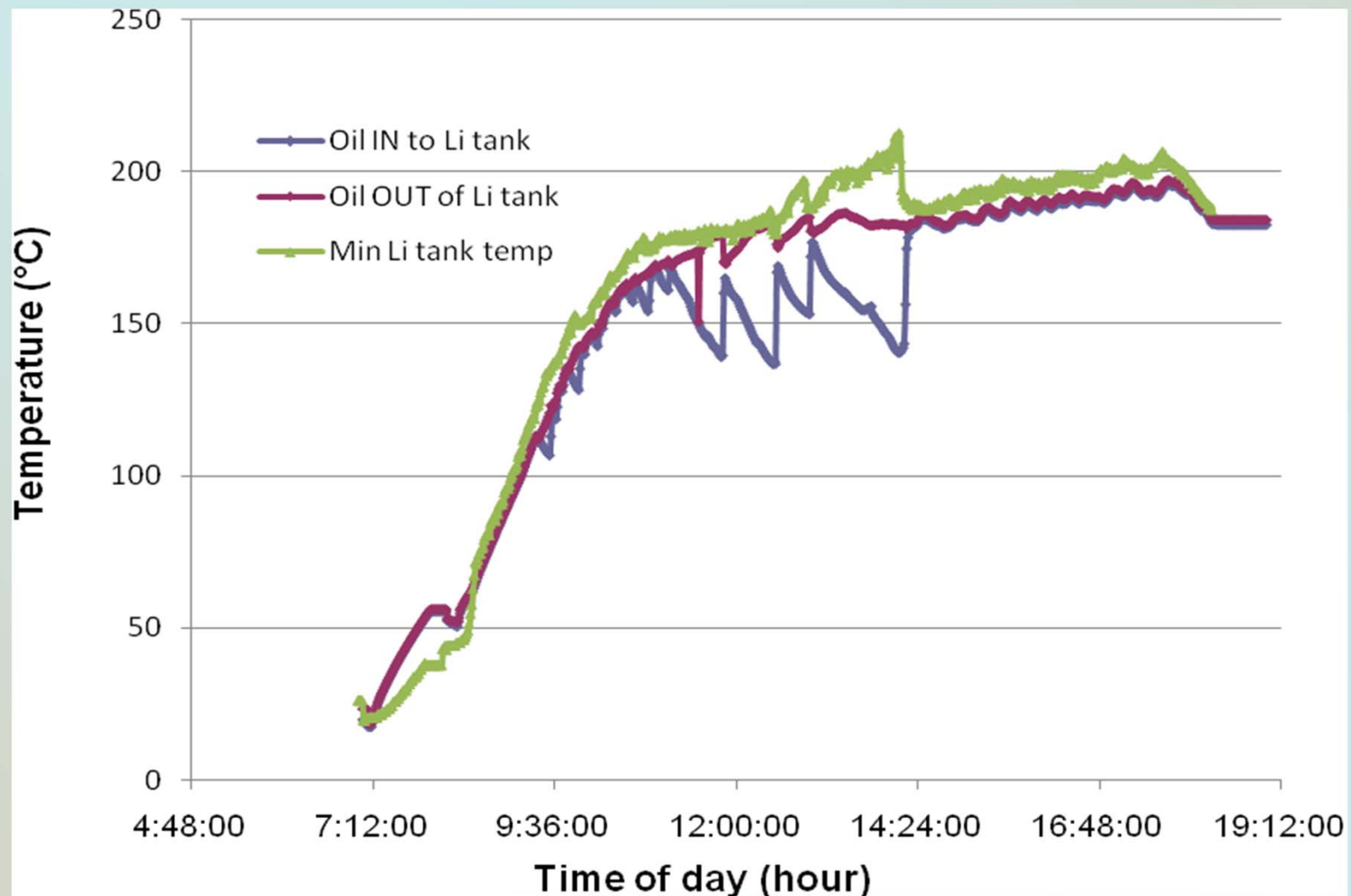
- ❖ Sm₂Co₁₇ permanent magnets: 12 units, 40x40x20 mm
- ❖ Operating temperature: up to 300 C
- ❖ Electrical Motor: Three Phase, 1.5 kW, 2800 rpm
- ❖ Variable Speed Motion Control: Three Phase, 1.5 kW
- ❖ Pump Dimensions: L= 700, D=350, H=320

- ❖ Loop sizes: OD 173.5 mm, width 20 mm, thickness 6 mm
- ❖ Magnetic Field at center: 3.2 kG
- ❖ Momentum Test: 115 N.m
- ❖ Calculated pressure: 8 At

Titanium adsorption vacuum pump



Oil temperature

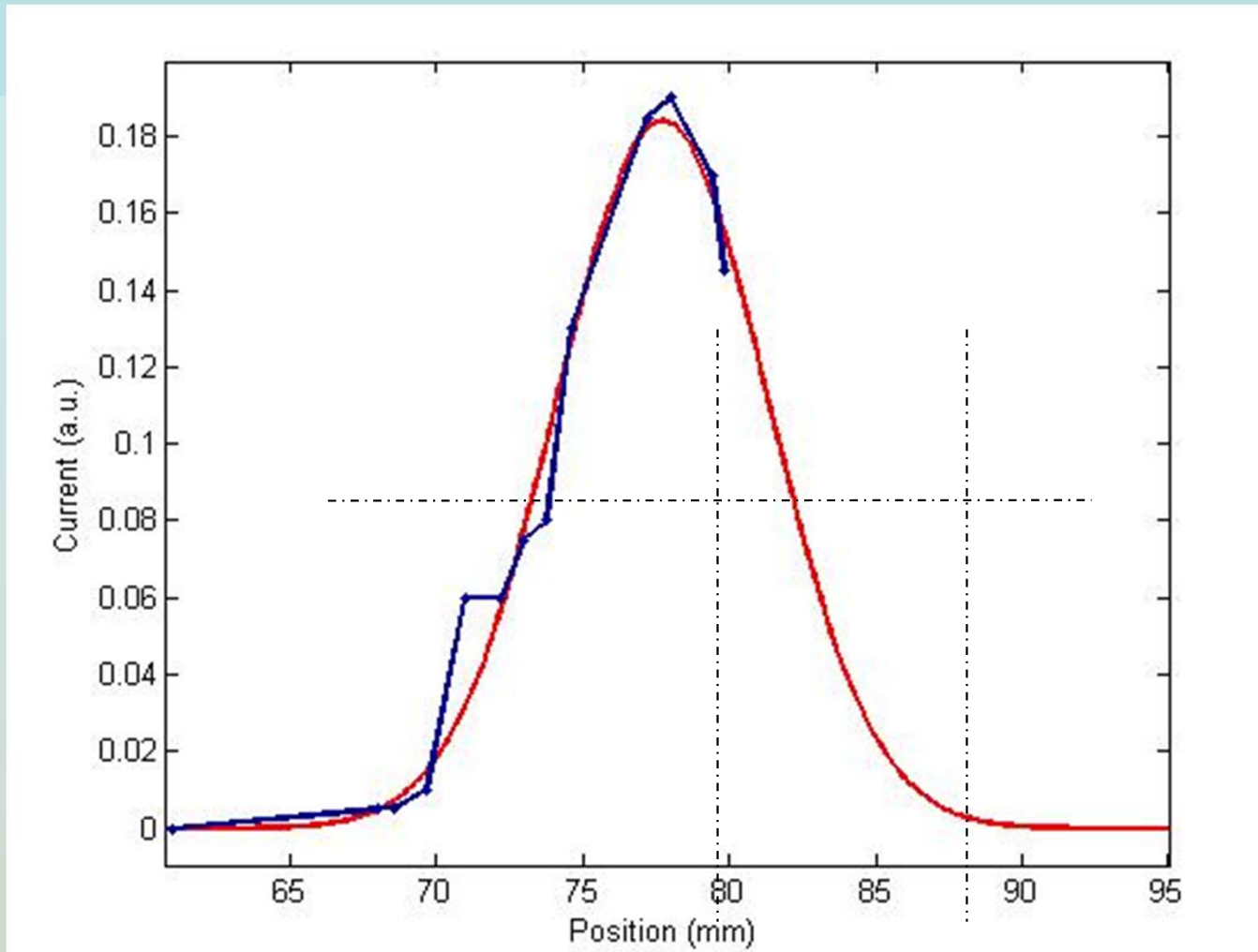


מהירויות של המשאבה EMP

טמפרטורה / הספק	הספק מקסימאלי (זרם mA) - kW	מהירות m/s	יחידות EMP 1-10 SPEED (%)	מס' הרצה
	1 (50)		2 (20%)	1- 4
1.2kW/615°C *	1.4 (70)	2.35	2 (20%)	5
1.2kW/614°C *	1.6 (80)	2.75	2.5 (25%)	6
1.2kW/571°C *	2.2 (110)	3.14	3 (30%)	7

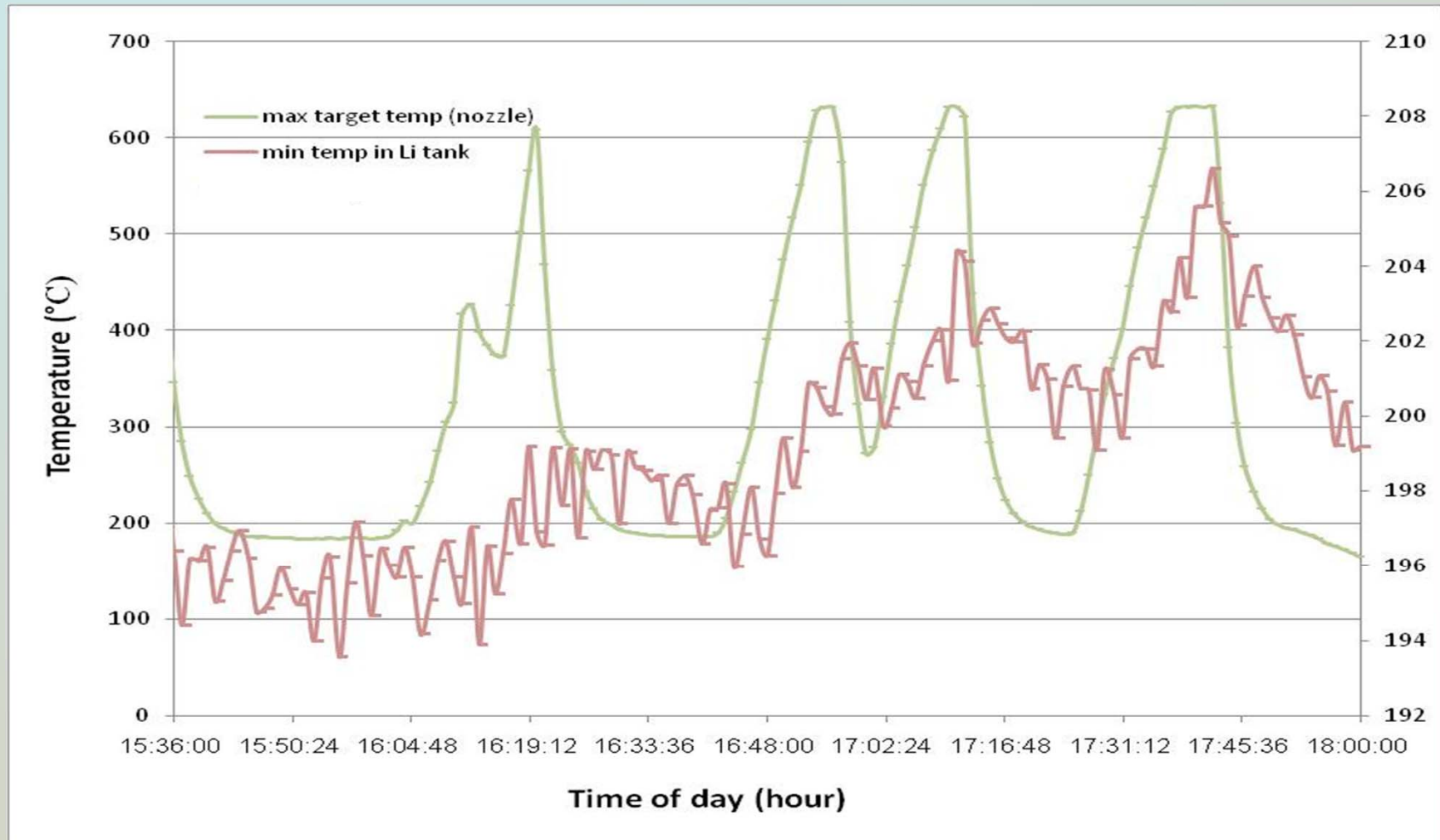
* הפרשי טמפרטורות ביחס למהירות זרימה הם על פי רישום ידני

צפיפות ההספק המקסימאלית בנסויי הינה 2.85 kW/cm^2 והצפיפות ההספק הנפחית

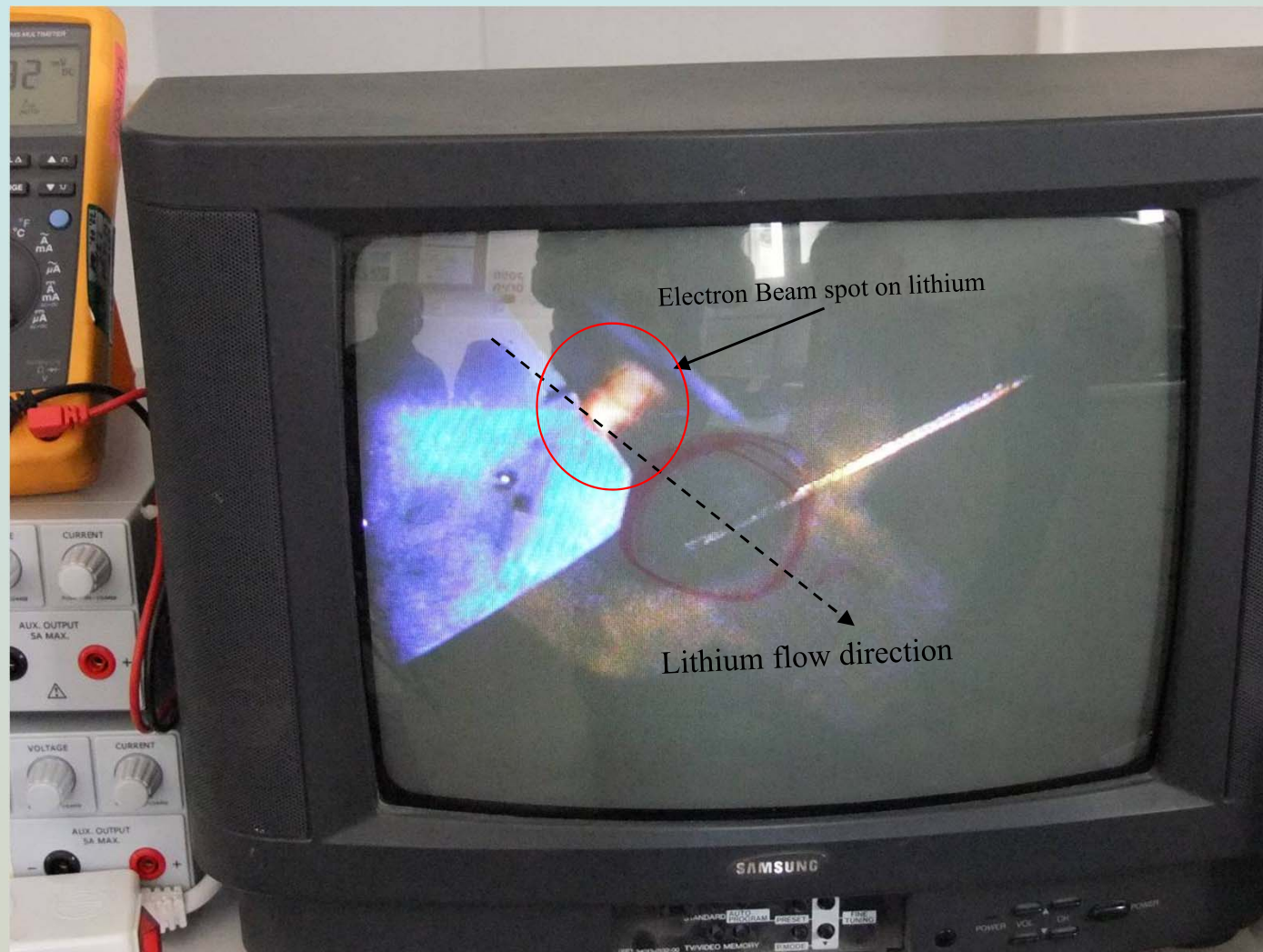


פרופיל קרן האלקטרונים בזרם של כ-10 mA (כחול) והתאמתם לגאוסייין (אדום) אשר מרכזו ב- 78 mm. בעל רוחב מחצית גובה של 8.3 mm וסיגמא של 3.5 mm

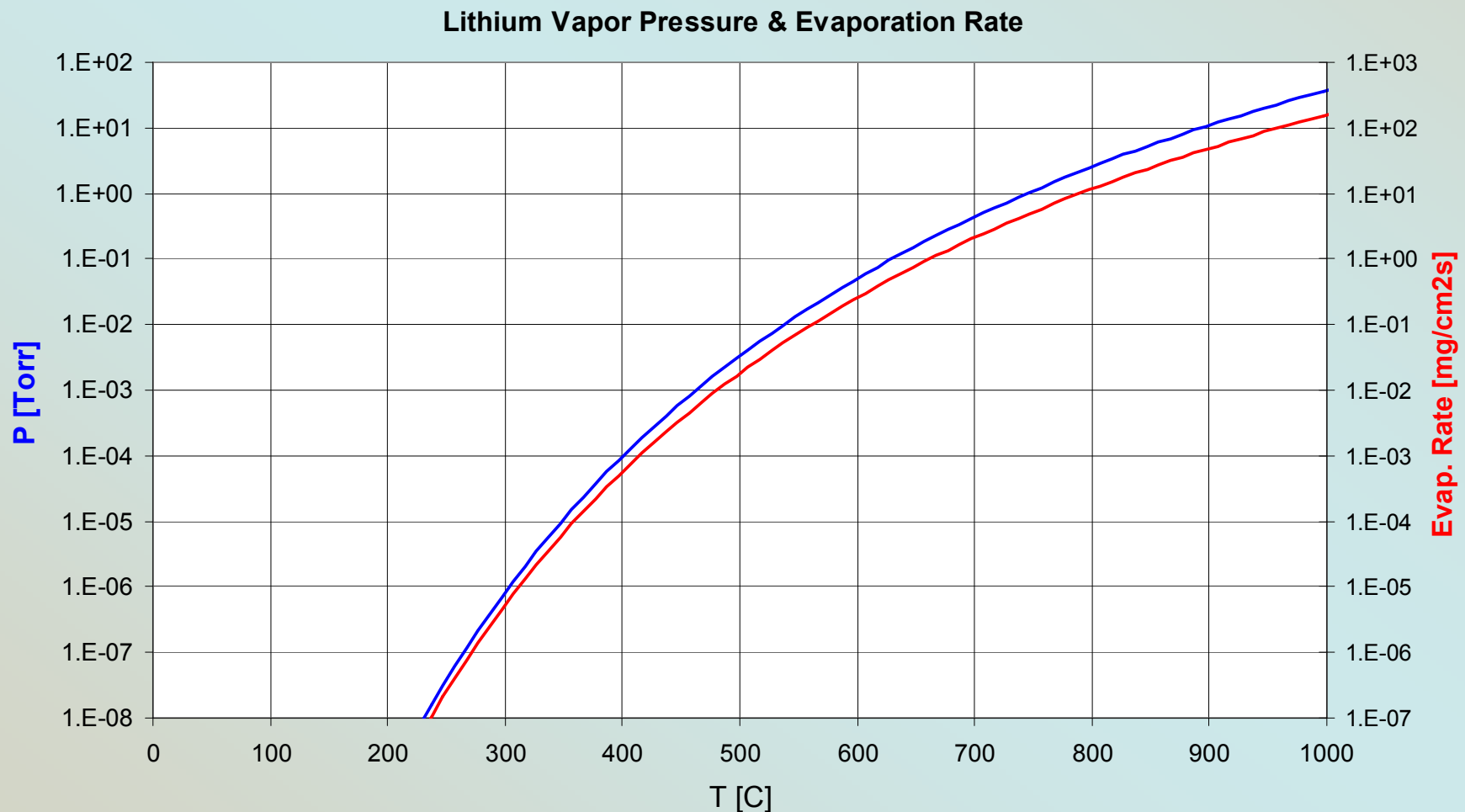
טמפרטורת הליתיום המינימאלית שנמדדה במיכל במקביל לטמפרטורת אוזני הנחיר במהלך ארבעת ההקרנות האחרונות בתותח האלקטרוניים

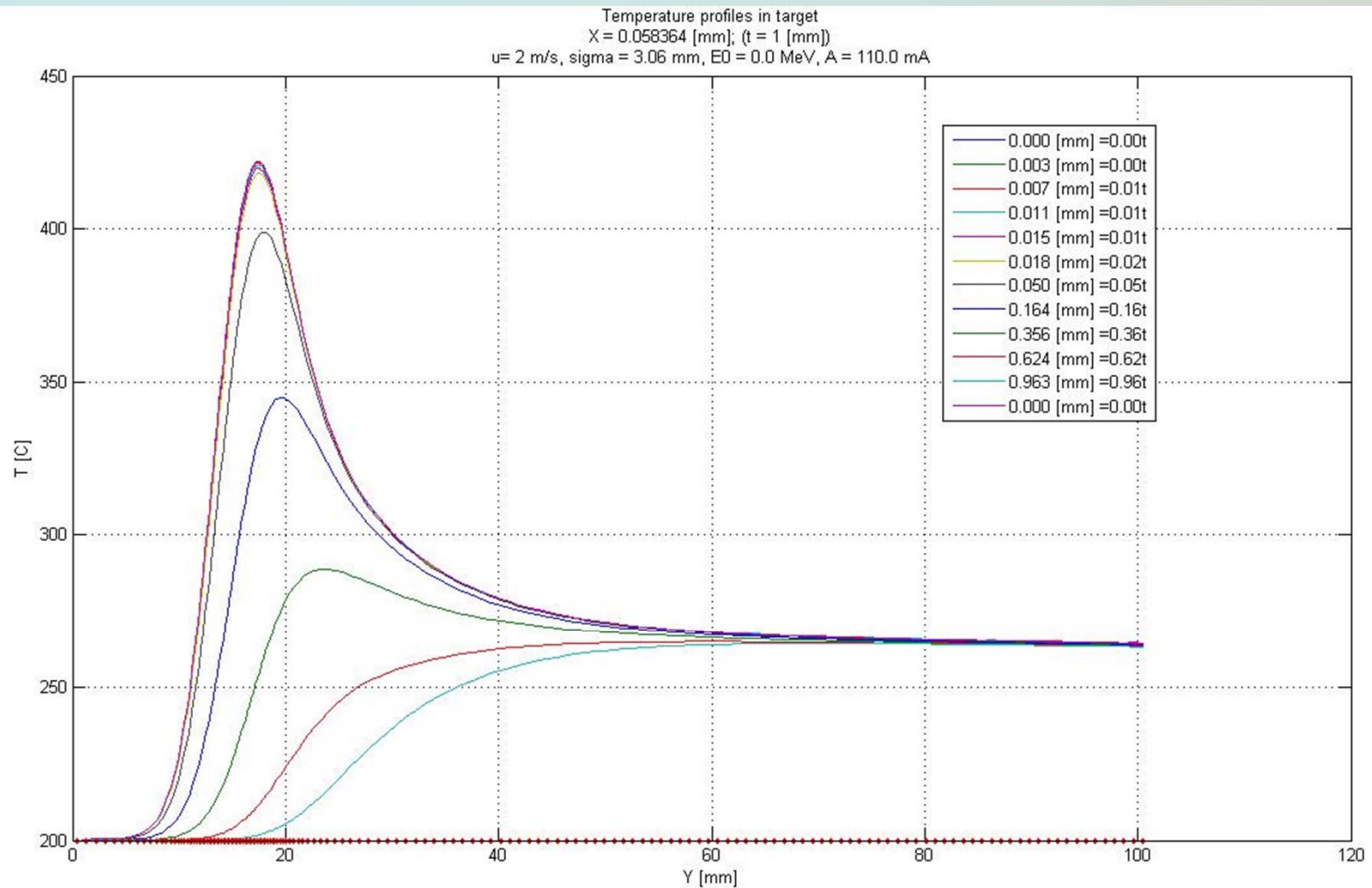


צילום הליתיום מסוחרר בנחיר בעת הקרנה בתותח אלקטרוני

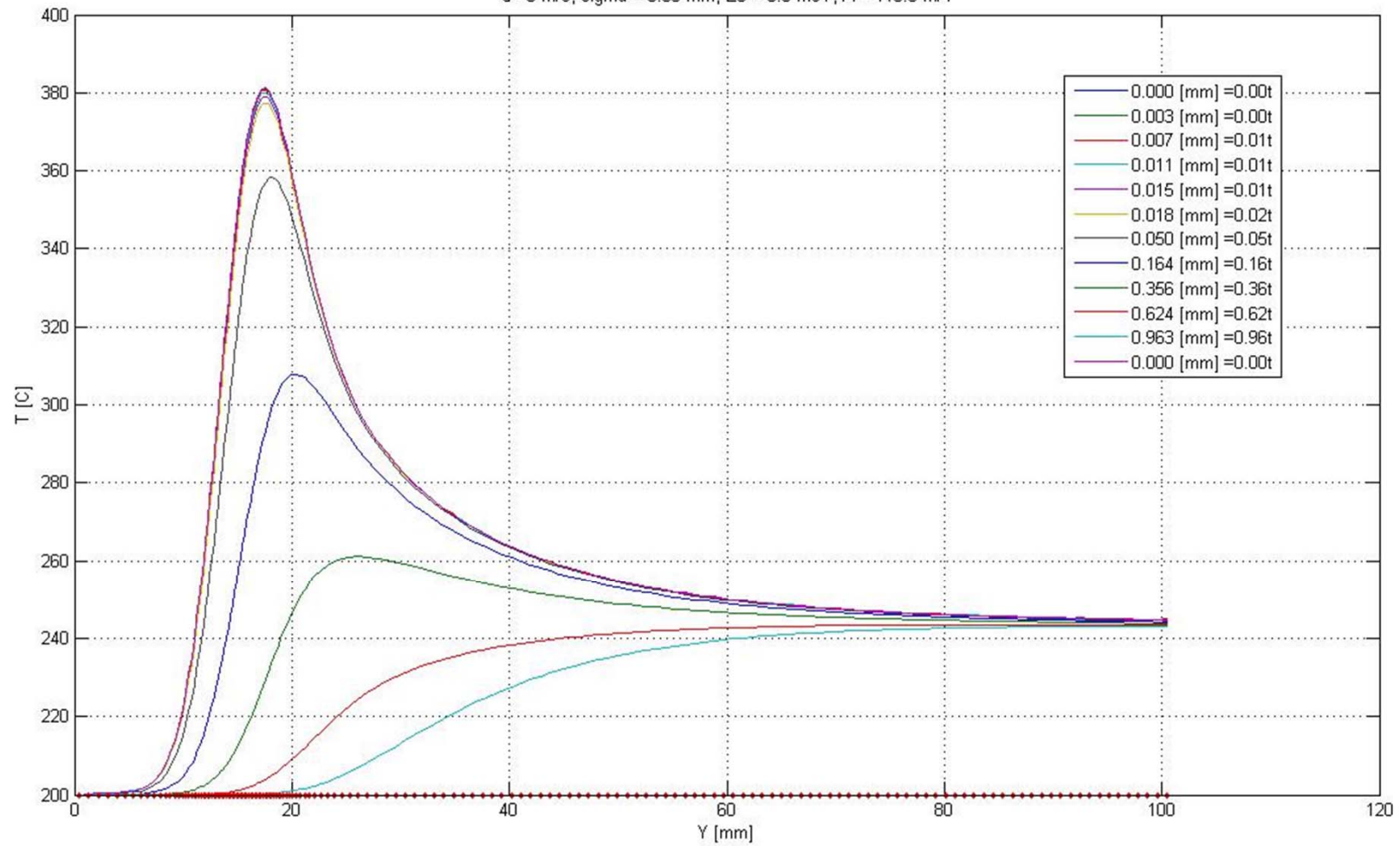


Lithium Vapor Pressure & Evaporation Rate





Temperature profiles in target
 $X = 0.058364$ [mm]; ($t = 1$ [mm])
 $u = 3$ m/s, $\sigma = 3.06$ mm, $E_0 = 0.0$ MeV, $A = 110.0$ mA



SARAF – Sores Applied Research Accelerator Facility

- ❖ To enlarge the experimental nuclear science infrastructure and promote the research in Israel
- ❖ To develop and produce radioisotopes primarily for bio-medical applications
- ❖ To modernize the source of neutrons at Soreq and extend neutron based research and applications