The JPARC Neutrino Target

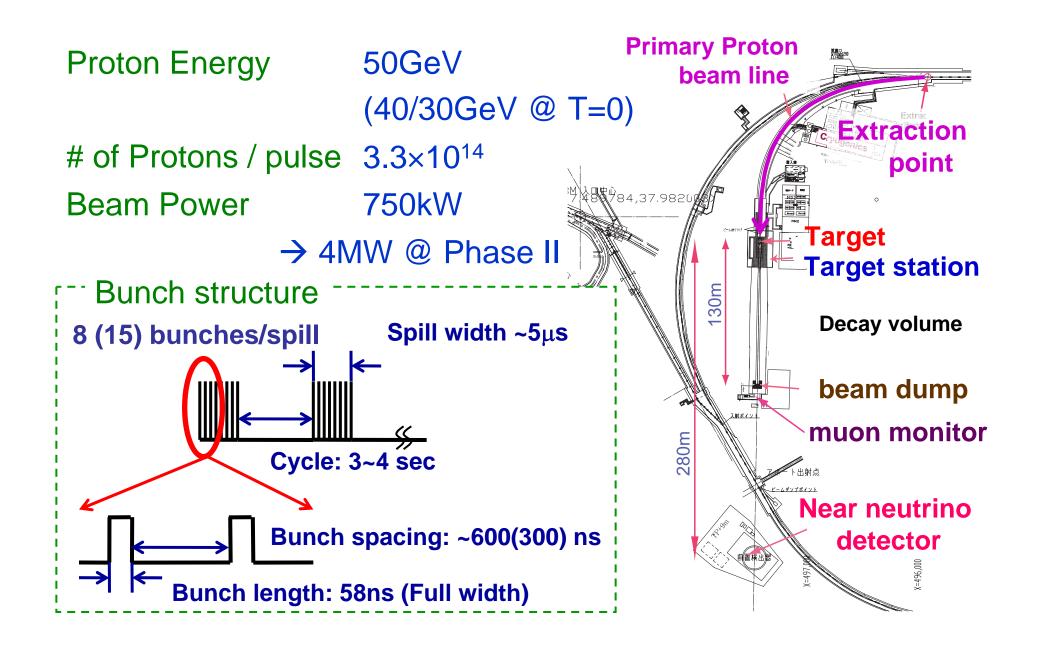
Y.Hayato (ICRR, Univ. of Tokyo)

for T.Nakadaira

& J-PARC v beam-line construction group

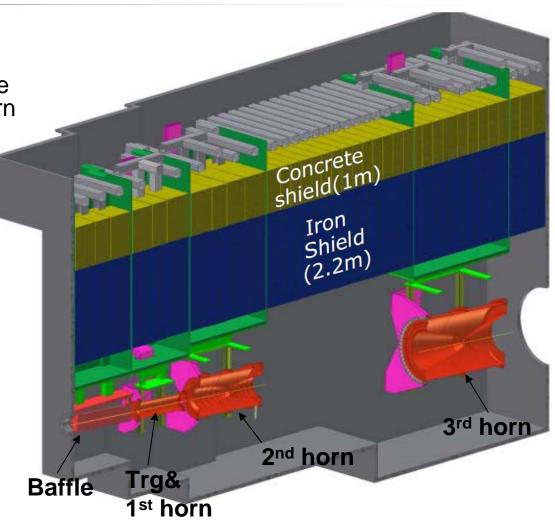
(I borrowed most of all the transparencies from Nakadaira-san)

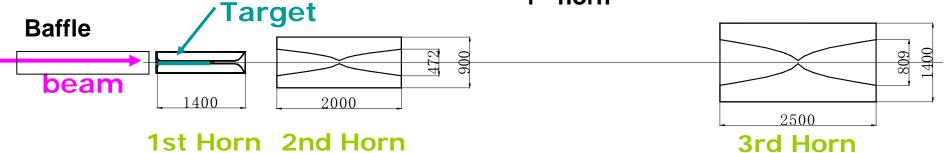
J-PARC neutrino beam line



Target Station

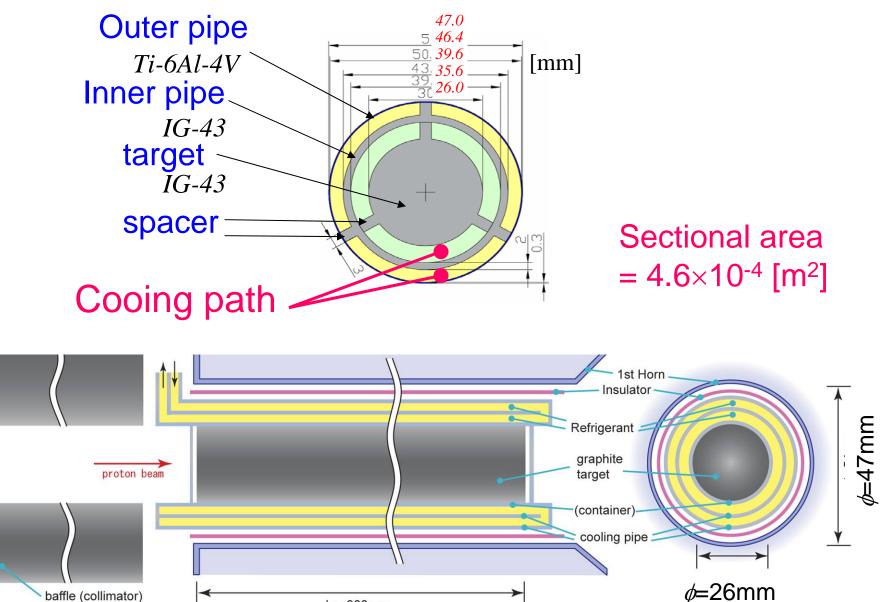
- Accommodate
 - Baffle: Graphite, 32mm
 hole x 1.7m long to protect 1st horn
 - Target
 - 3 Horns
- Area filled with Helium gas
 - reduce Tritium, NOx production
- Highly radio-activated
 - _ ~1Sv/h,
 - Need remote-controlled maintenance system
- Need cooling (Helium vessel, radiation shield,..)





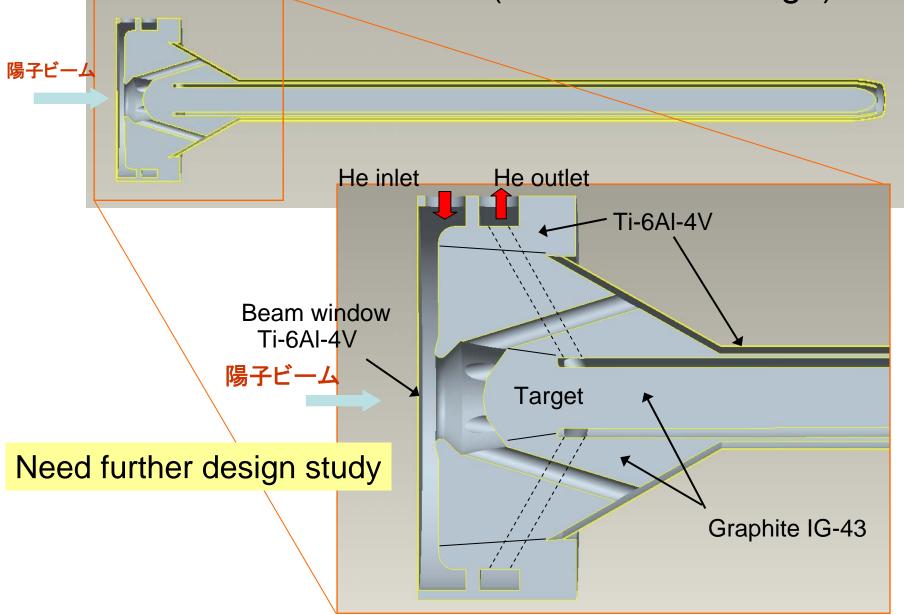
Target dimension

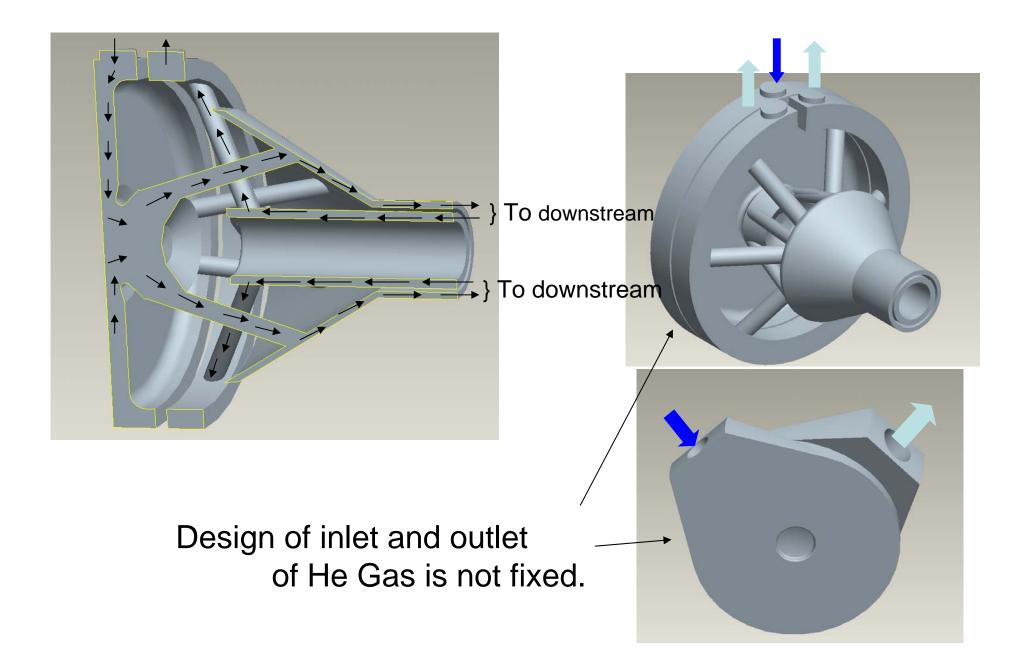
• Co-axial 2 layer cooling pipe: Graphite / Ti-6AI-4V, Helium cooling



L = 900mm

One design of the target & surrounding cooling pipe (Not the final design).



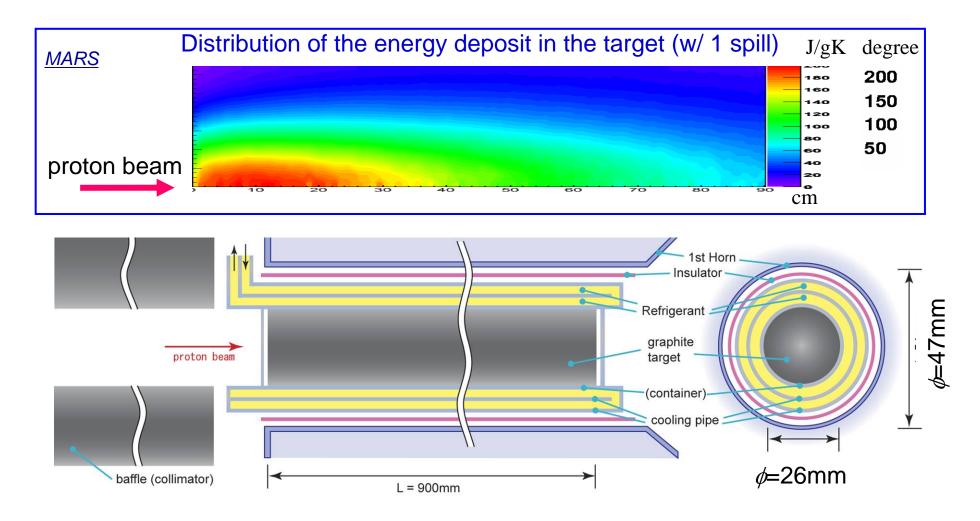


Target cooling : Water or Helium?

	Water Cooling	Helium Gas cooling
Advantage	 ○Very Efficient ← High Heat transfer ratio ○ Already tested. ○ Simple circulation system 	 We can control T_{target} to minimize the irradiation effect. (T_{target} = 400 ~ 800 °C) Reduce radioactive waste water No target container is needed
Disadvantage	 Iarge Irradiation effect ← T_{target} ≤ 300°C to avoid the water boiling: T_{target} @ surface < 100°C Target container is needed. ΔP_{water}= ~2MPa due to the temperature rise by a beam hit. Huge radioactive waste water 	 Very High flow rate ~ 2000 I/min [0.5MPa] ~ 12000 I/min [0.1MPa] Circulation system is complicated. Special treatment for the high temperature gas (200°C) is necessary.

Target : Conceptual design

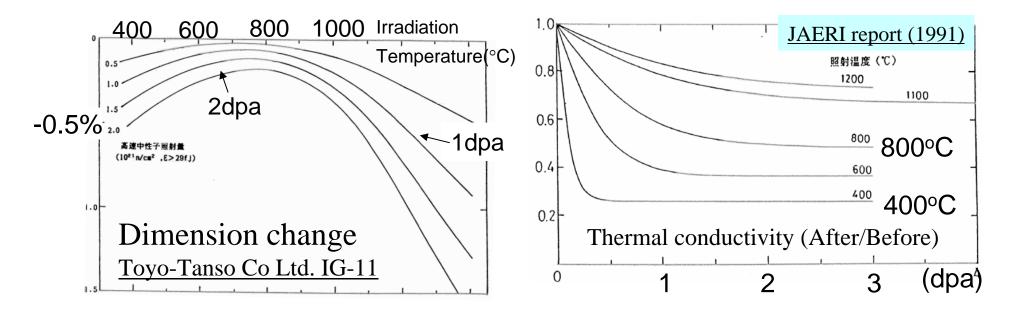
- Core: Isotropic-Graphite : IG-43 (Toyo Tanso Co. Ltd)
 - − Energy deposit... Total: ~40kJ/spill (30GeV)
 → ΔT ≈ 200K. σ_{eq} = 7.5 [MPa] ↔ Tensile strength = 37.2 [MPa]



R target	13 [mm]	
ΔR (target ~ horn)	3[mm]	
Energy deposit (30GeV, $\sigma_x = \sigma_y = 4.24$ mm)		
Target	39.3 [kJ/spill]	
Inner Pipe	3.5 [kJ/spill]	
Outer Pipe	1.1 [kJ/spill]	
Insulator	1.5 [kJ/spill]	
Targeting Efficiency	99.09%	
For Gaussian beam	(0.91%loss)	
Helium flow (T _{gas} < 200°C, suction=0.03MPa)		
Cross section	459.3 [mm ²]	
Flow rate	491 [Nm³/h]	
Avg. speed @ target	246 [m/s]	
$\Delta {\sf P}$ @straight part+1 st hex	0.0833 [MPa]	

Irradiation Effect of Graphite

- Expected radiation damage of the target
 - The approximation formula used by NuMI target group : 0.25dpa/year
 - MARS simulation : 0.15~0.20 dpa/year
- Dimension change : shrinkage by ~5mm in length in 5 years at maximum.
 ~75μm in radius
- Degradation of thermal conductivity ... decreased by 97% @ 200 °C 70~80% @ 400 °C
- Magnitude of the damage strongly depends on the irradiation temperature.
 - It is better to keep the temperature of target around <u>400 ~ 800 °C</u>

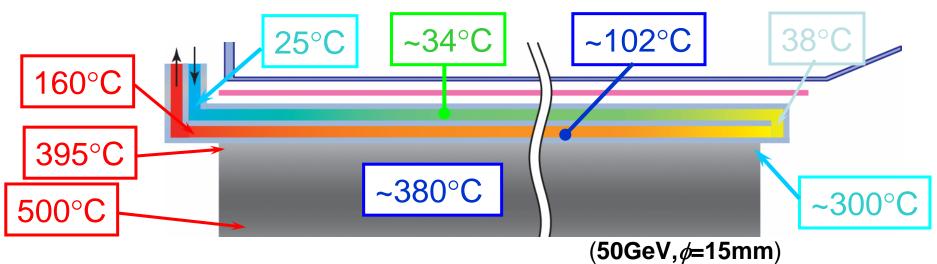


FEM simulation of He cooling

 Assumptions: <u>0.19MPa</u> He Initial temperature: 25 °C He flow rate: 6000 [l/min] → 194 [m/sec] Heat convection rate: 820 [W/m²/K]

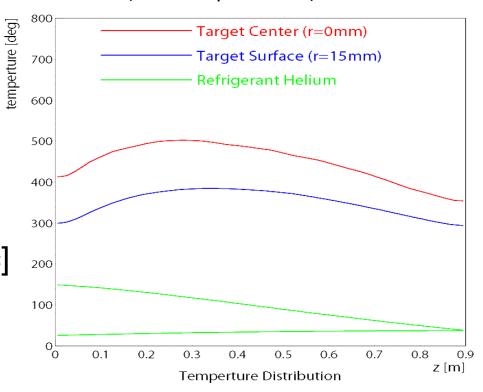
Temperture (deg) 600 250 T_{max}(inner) F_{avg}(inner) T_{max}@surface T_{ava}@surface max (outer) 500 200 T_{max} (Equilibrium) = 158.87 T_{max} (Equilibrium) = 37.83 -avg (cquilibrium) = 33.64 400 150 300 100 200 50 T_{max} (Equilibrium) = 502.50100 T____@surface (Equilibrium) = 394.08 0,0 0 50 100 150 200 250 30(50 100 150 200 250 300 t(sec) t(sec) **Target Temperature Helium Temperature** Max: ~ 500 °C Min: ~ 300 °C Max: ~160 °C (ΔT = 135 °C) Avg. : ~380 °C Avg. : ~40 °C

FEM simulation of He cooling



- Assumptions: <u>0.19MPa</u> (←Possible pressure drop at the downstream of target is taken into account.)
- He Initial temperature: 25 °C He flow rate:

6000 [l/min] → 194 [m/sec] Heat convection rate: 820 [W/m²/K]



fin.