

Calculations of the radiological environment for handling of ISOLDE targets



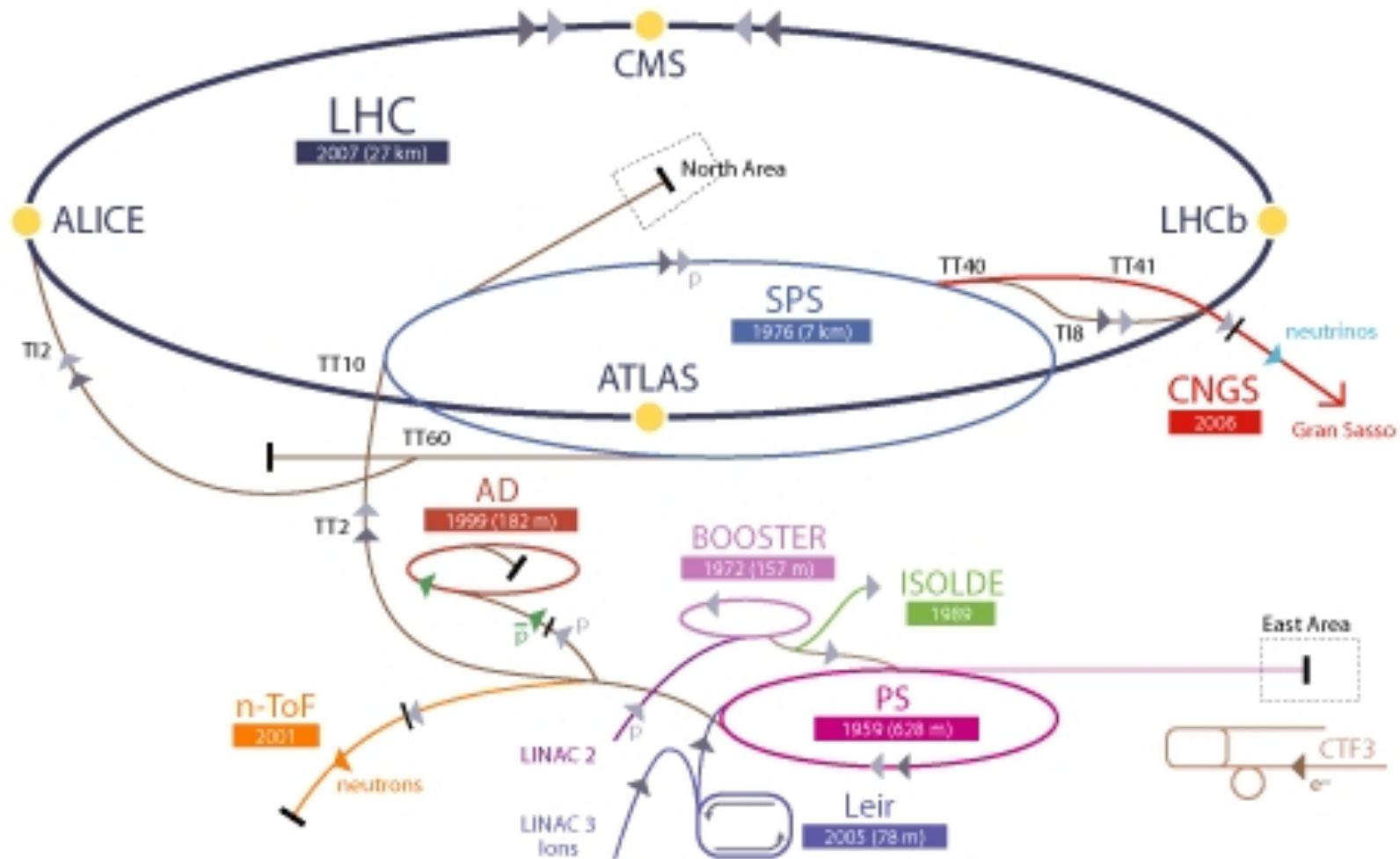
HSE
Occupational Health & Safety
and Environmental Protection Unit

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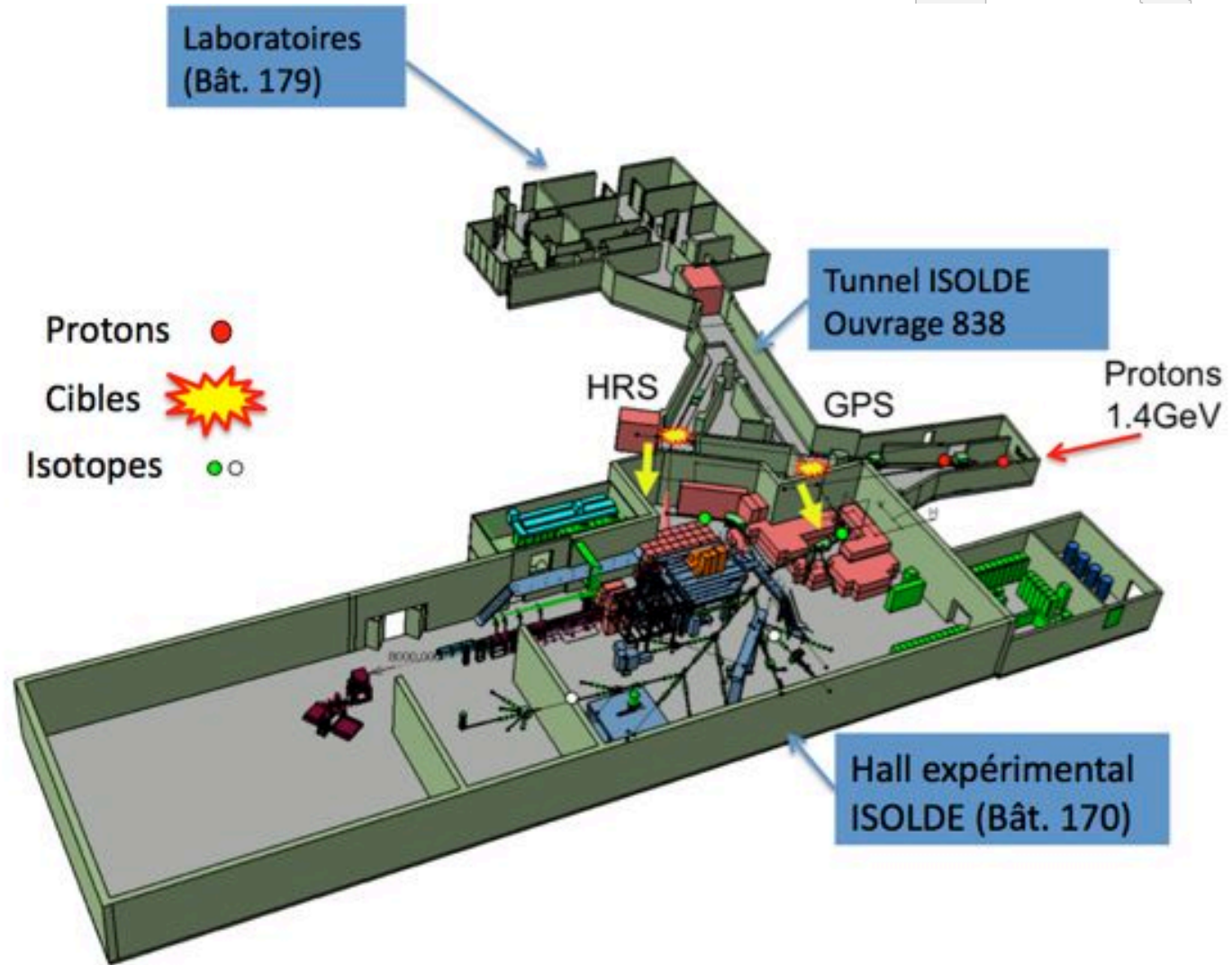
Outline

- ▶ Short description of ISOLDE
- ▶ ISOLDE target handling system (replacement project)
- ▶ Results of FLUKA calculations
- ▶ Conclusions and perspectives

CERN accelerators complex

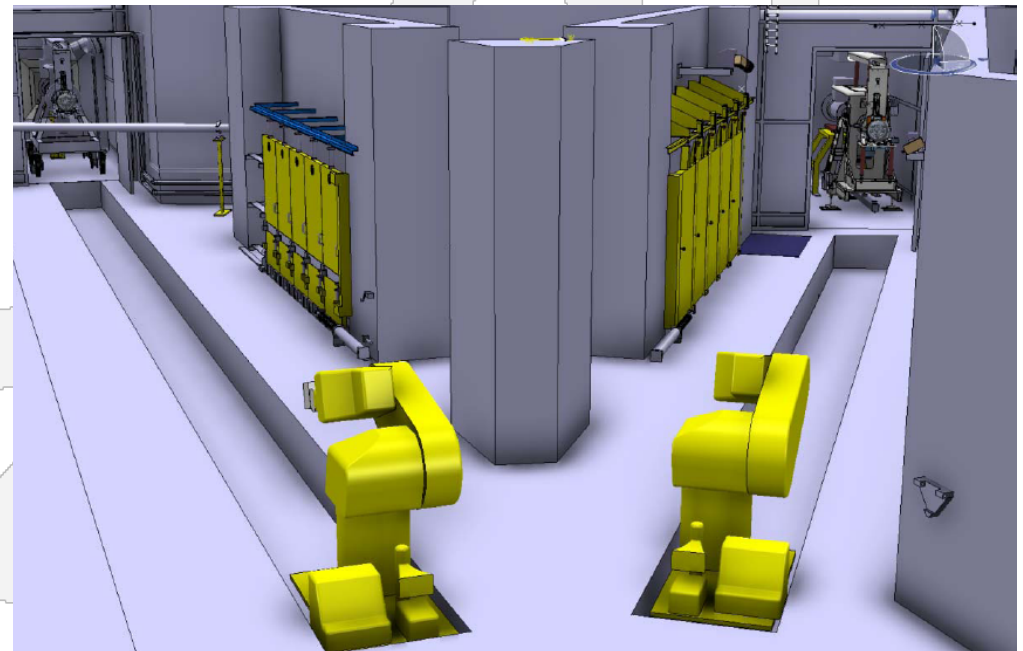


General layout of the ISOLDE facility



Current target handling system

- ▶ Two robots mounted on rails (located in a trench)
- ▶ Removal of targets and transportation to intermediate storage shelves and installation of the new target
- ▶ ~30 target exchanges performed per year



Target storage and hot cell project

- ▶ Used targets transferred to a temporary storage area prior to maintenance activities during the shutdown
- ▶ Project to build a hot cell for target dismantling and conditioning prior to final storage.

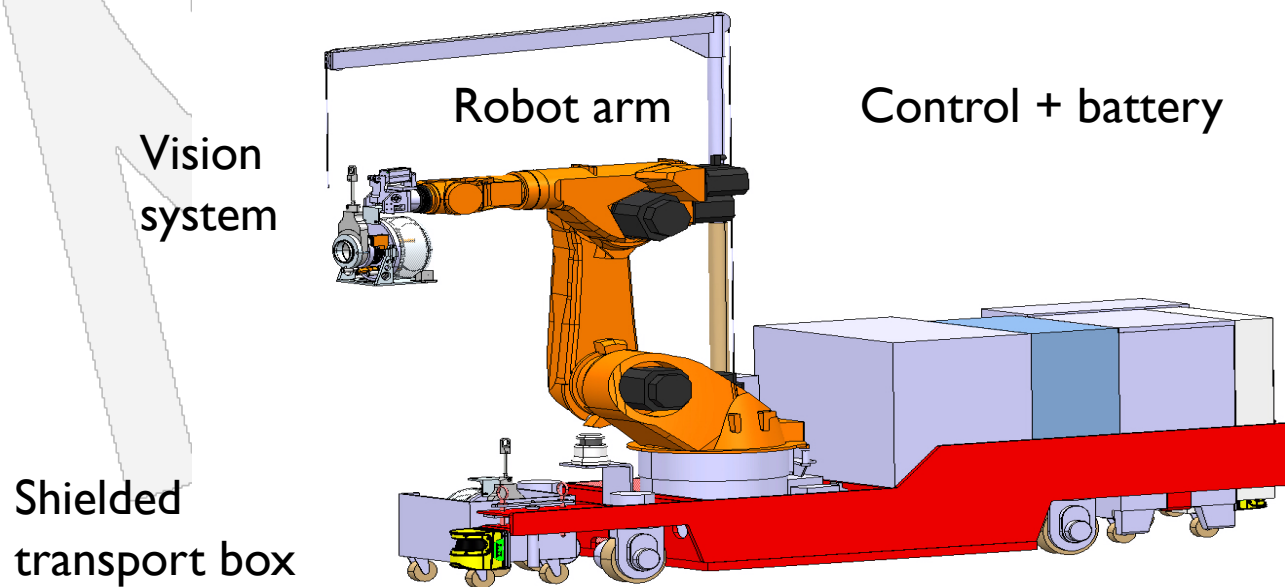


Robot replacement project

- ▶ Current system arriving at the end of its lifetime and different concepts being investigated
 - ▶ One autonomous Automated Guided Vehicle (AGV) parked outside the target area during operation
 - ▶ Current concept (2 robots on rails) with some improvements
- ▶ Both approach have pros and cons: maintenance aspects, reliability, recovery procedure and flexibility have to be taken into account in the choice of the two concepts
- ▶ Monte-Carlo simulations (FLUKA) needed to assess the radiation environment in which the robot will evolve (radiation hardness, recovery scenario...)

Automated Guided Vehicle concept

- ▶ **Automated Guided Vehicle (AGV):**
 - ▶ Fully autonomous vehicle
 - ▶ Integrated robot arm
 - ▶ Robot mounted vision system for precise robot control



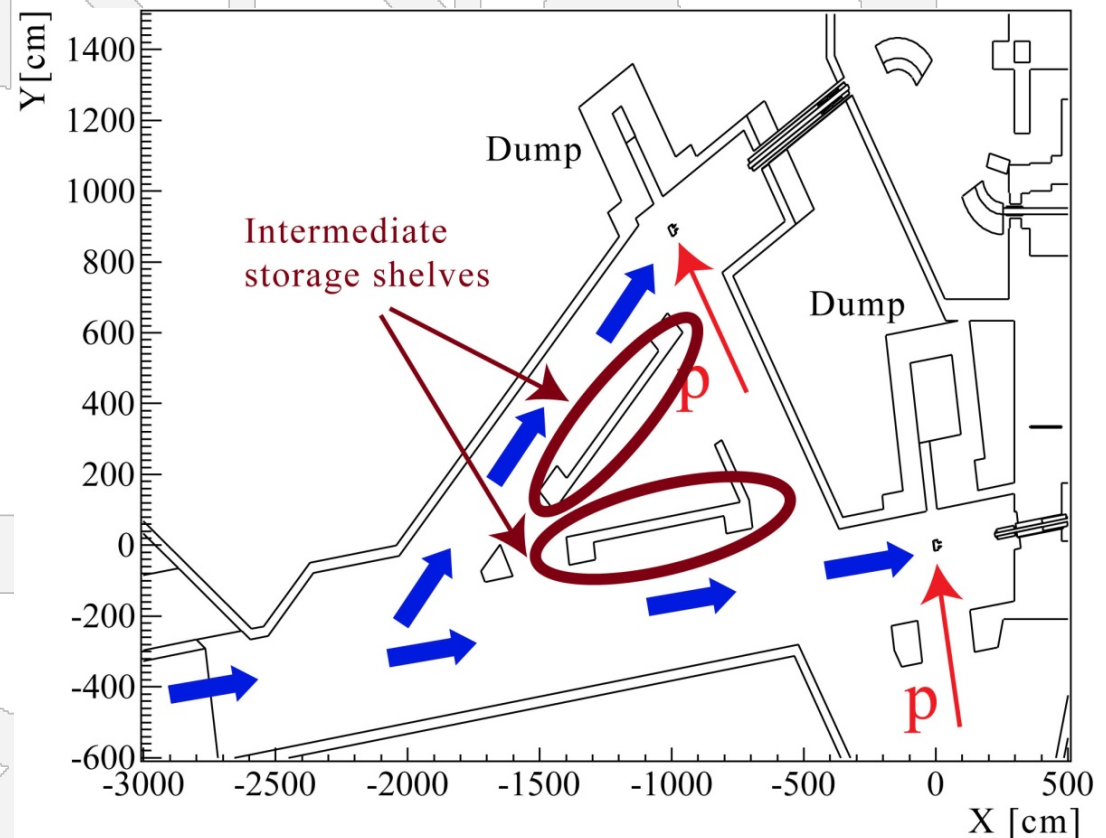
Radiation impact and constraints (AGV)

- ▶ Robot exposed to residual dose rate during target exchange:

- ▶ Dose to vision system
- ▶ Recovery

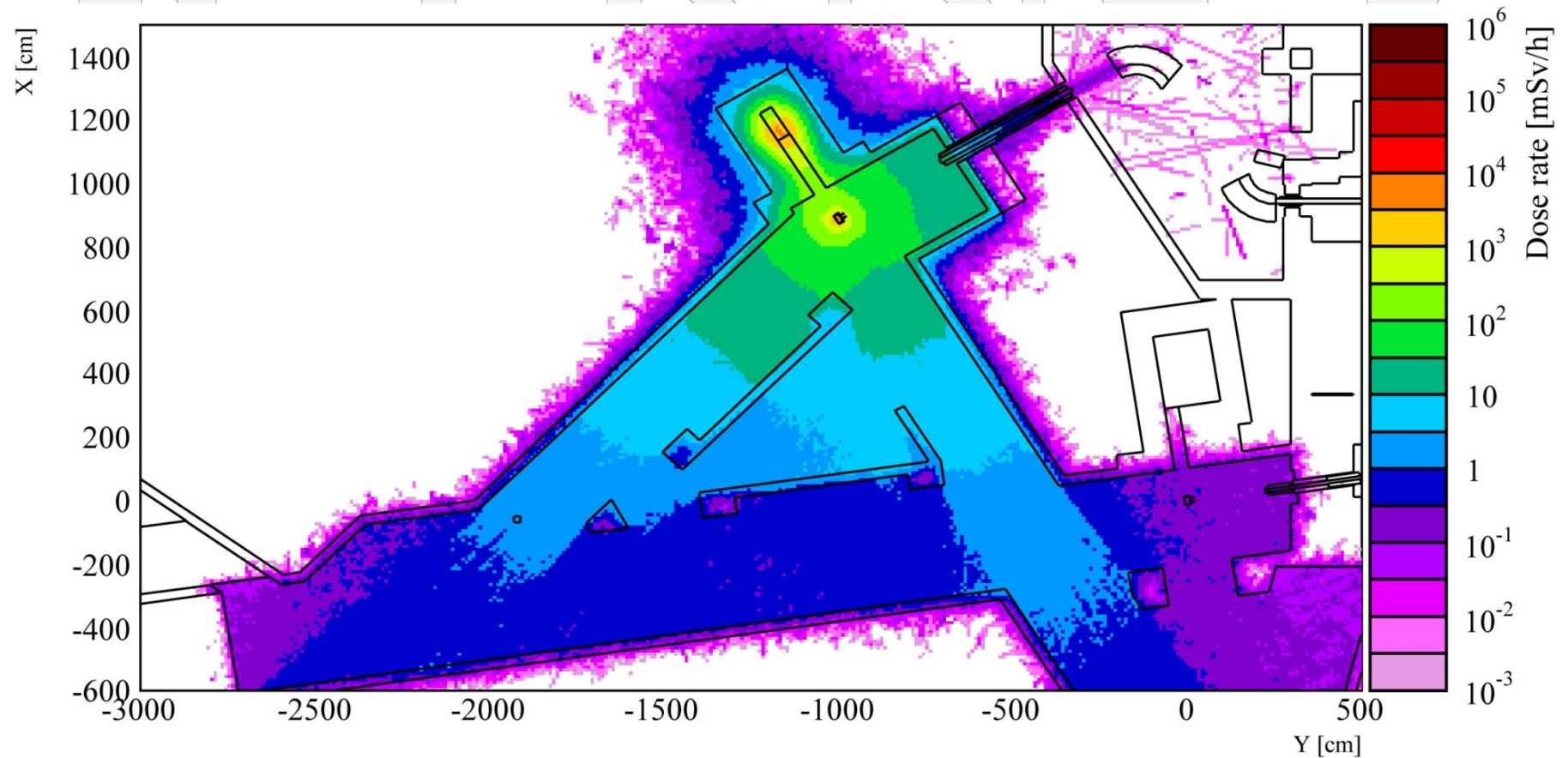
- ▶ 1×10^{19} protons over 220 h of operation
- ▶ UC target considered
- ▶ Different cooling times considered
- ▶ Cooling before target exchange = 3 days

FLUKA geometry of the area

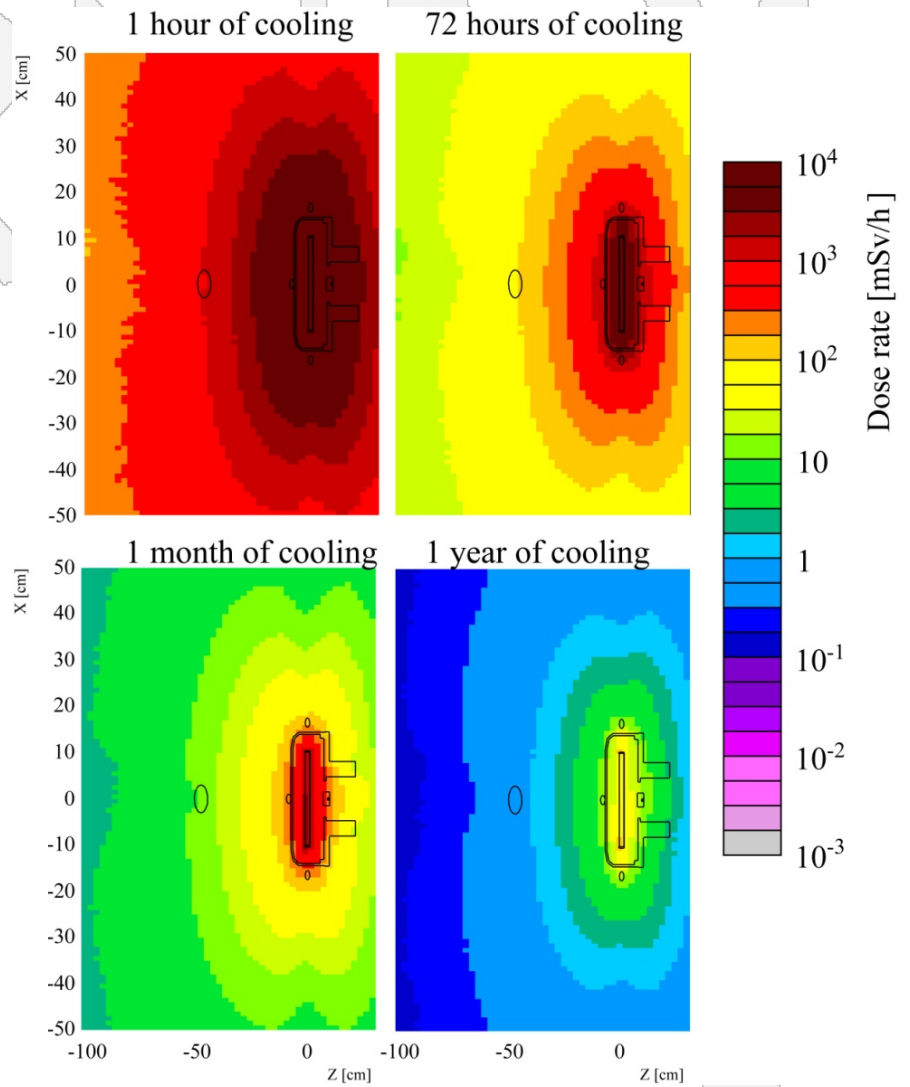
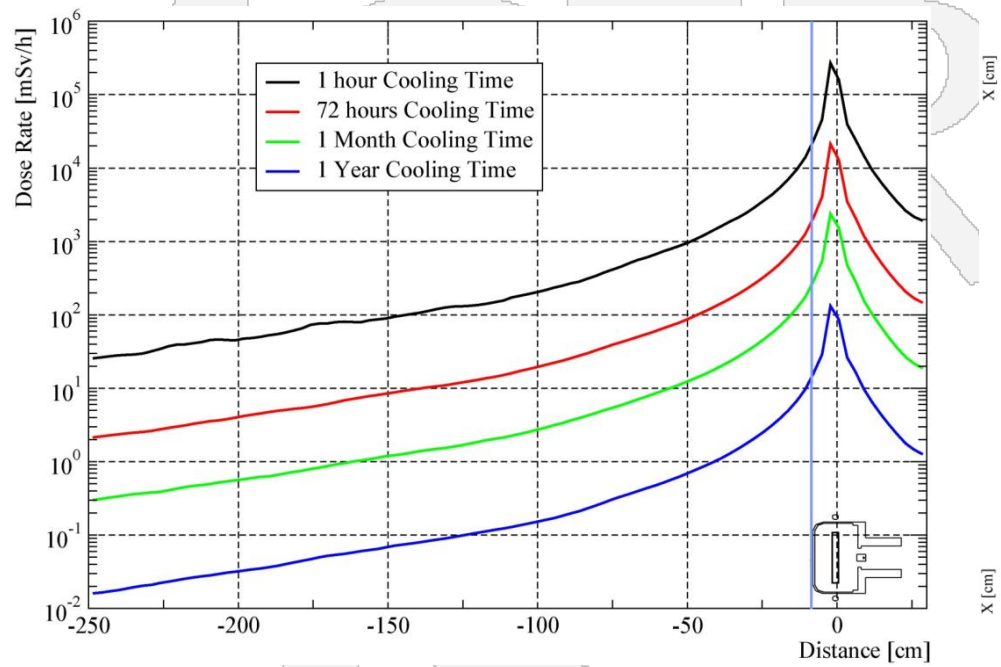


Example of residual dose rate map (72 h CT)

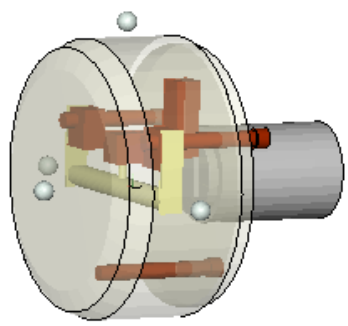
- ▶ Most of the beam absorbed in the beam dump (shielded)
- ▶ Several Sv/h on contact with the target (after 72 h of decay)
- ▶ No access above 100 mSv/h



Close view of the target environment

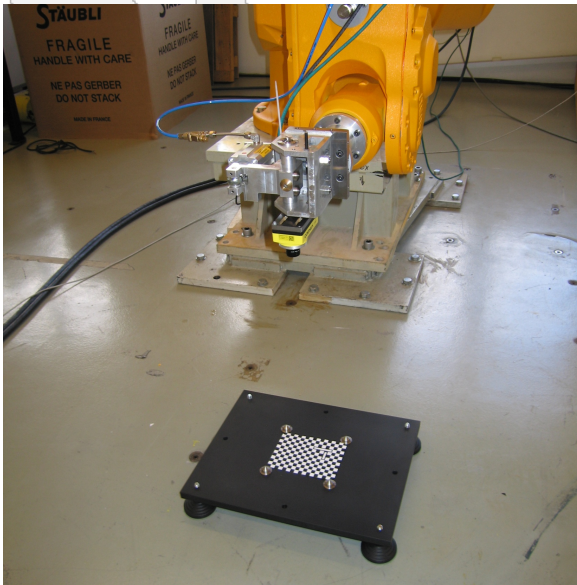


Gray/h close to the target from residual radiations

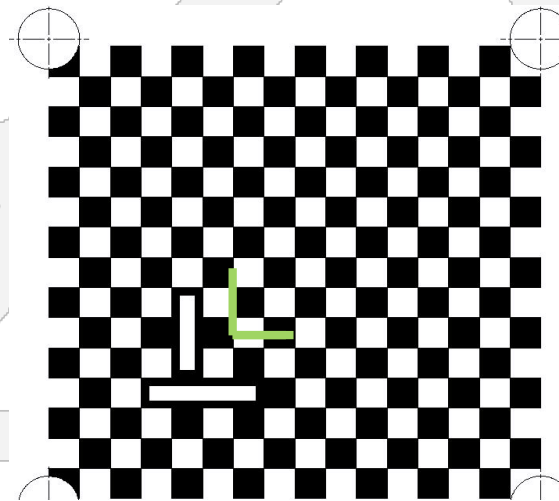


Dose to the vision/positioning system

- ▶ Vision system used for accurate robot positioning directly linked to system performance and reliability
- ▶ Envisaged CCD camera performance must be tested in similar radiation environment (illuminated exposure)
- ▶ To consider instantaneous and integrated dose effects

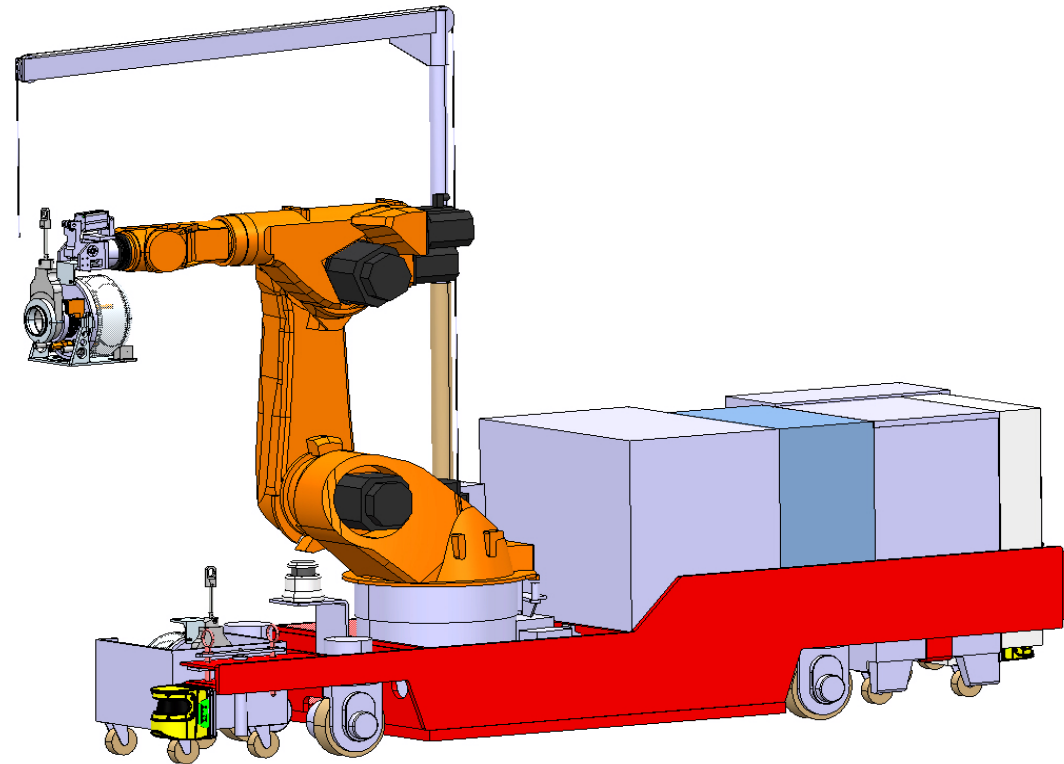
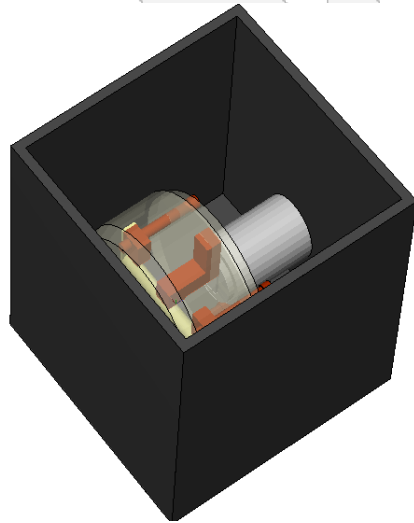


Camera calibration



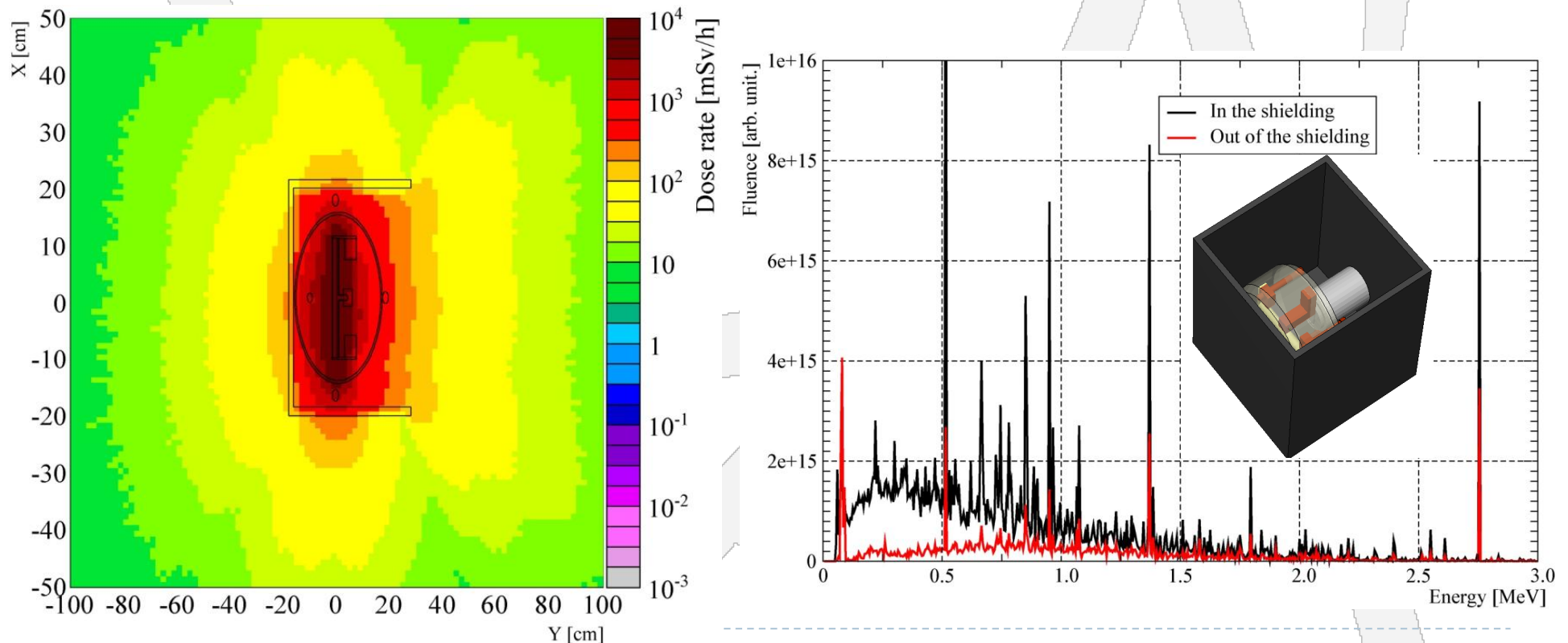
Benefits from the shielded transport box

- ▶ Objective is to minimize the time during which used targets are handled with the robot gripper
- ▶ Transport box can be disconnected from AGV in case of failure

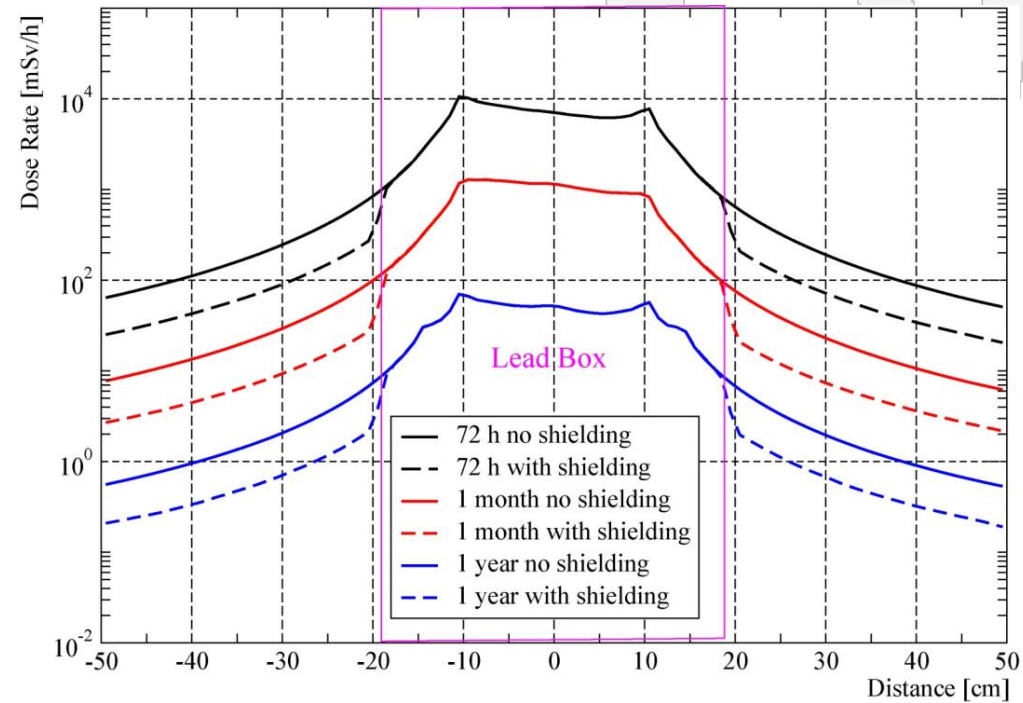
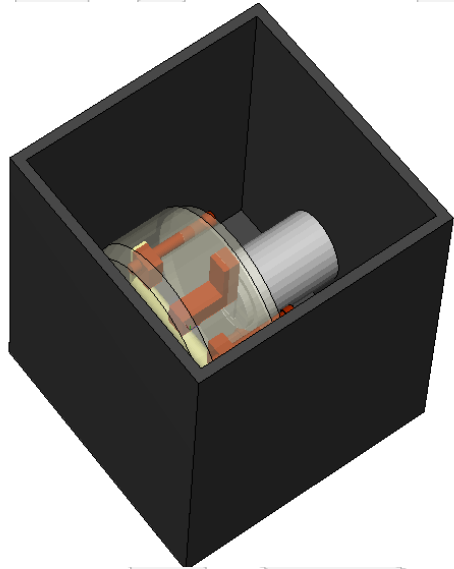


Calculations for the transport box

- ▶ Using a two steps approach where radioactive nuclei are “stored” in the first step and decay products transported in a different geometry during the second step



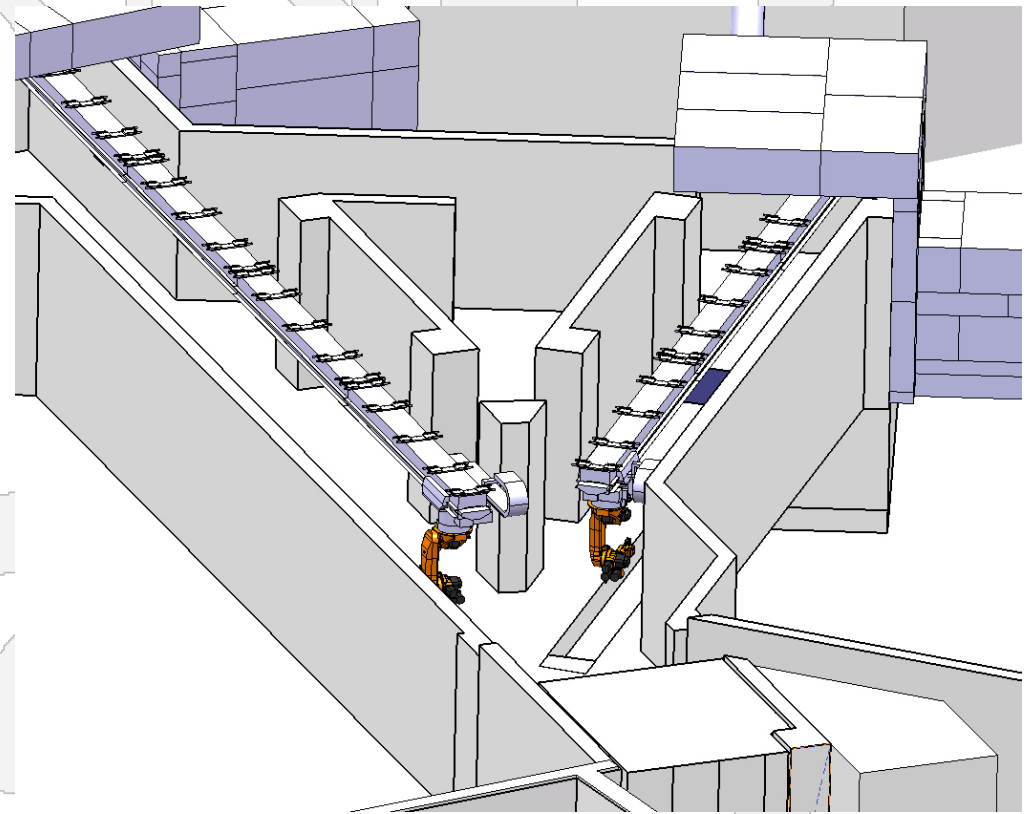
Residual dose rate for different CT



- ▶ Recovery could be complicated in case of failure (risk for operation - only one robot available)
- ▶ Dose to CCD camera for positioning system critical
- ▶ System is becoming quite complex and massive....

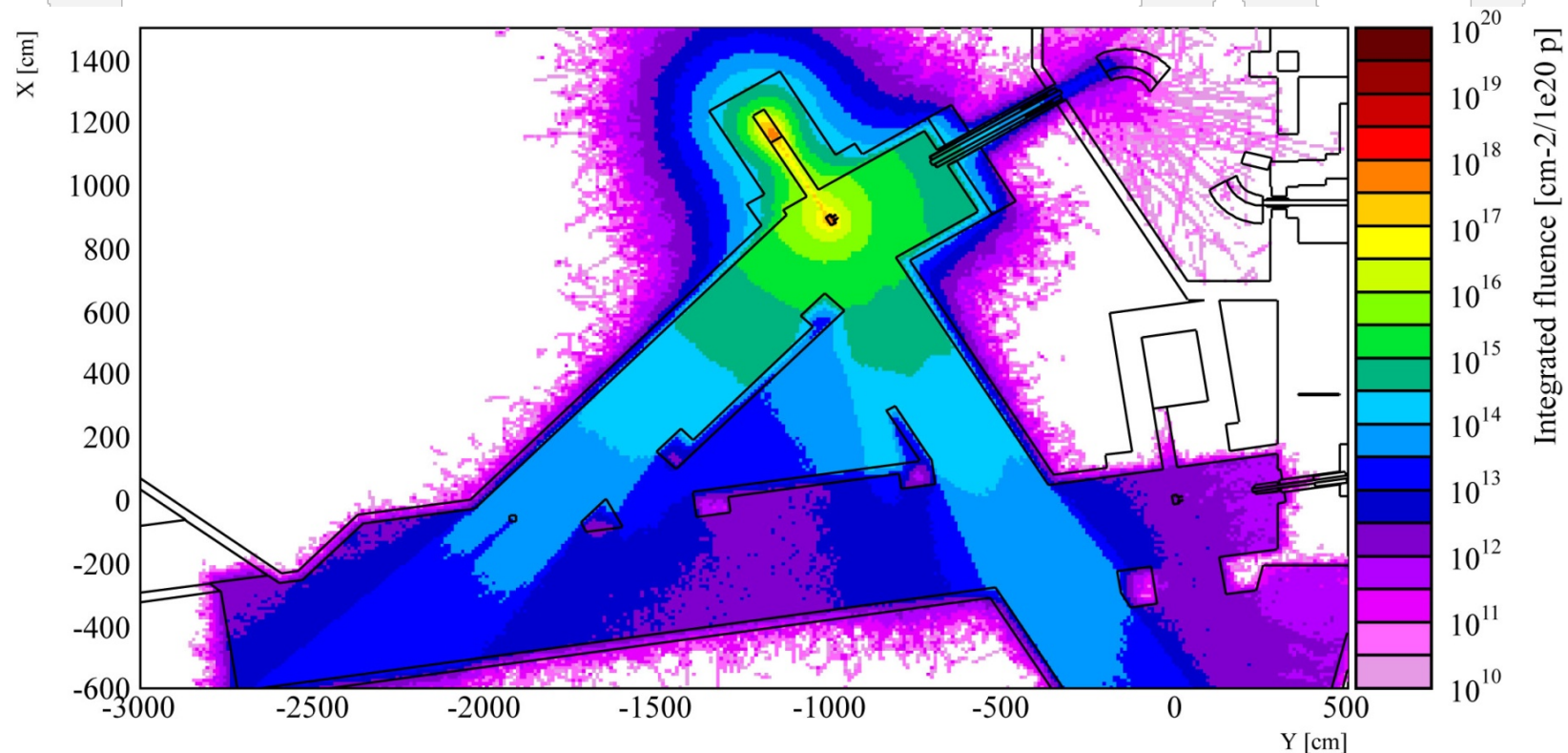
Alternative choice

- ▶ Similar system as the current one, except that the rail is mounted on the ceiling (avoiding trenches on the floor)
- ▶ Parked in the area during operation
- ▶ Controls located outside
- ▶ Benefits from current technology
- ▶ Sensitivity to radiation damage ?
- ▶ Calculation of radiation environment (cumulative effects)



Calculation for the most exposed robot

- ▶ Scoring the 1 MeV neutron equivalent fluence (probability of displacement damage in Si due to NIEL)
- ▶ Scoring of dose (deposited energy) for organic components
- ▶ Results used for the technical specification



Consideration for radiation hardness

- ▶ Integrated dose for organic material of the order of kGy per year (calculations results in agreement with TLD passive dosimetry measurements). Important for festoon cables.
- ▶ 1 MeV neutron equivalent fluence considering planned accelerator upgrade of the order of 10^{14} n/cm² per year
- ▶ Recommended to minimize electronic components inside the primary area (better for maintenance purpose as well)
- ▶ No issue with Single Event Upset as robot is off during beam operation

Summary and Conclusions

- ▶ FLUKA calculations of the residual dose rate around ISOLDE targets performed in the frame of the ISOLDE robot replacement project
- ▶ AGV option seems to be complicated (vision/AGS/robot) and not optimal from a reliability point of view (one robot + long cooling time for recovery)
- ▶ Similar concept as the one used today seems to be preferred with some improvements
- ▶ FLUKA calculations very useful for other target handling aspects (hot cell project for used targets dismantling, isotope inventory for risk analysis....)

Aknowledgements

- ▶ R. Luis and Y. Romanets for providing the FLUKA geometry used for those calculations